

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Dresden Nuclear Power Station, Unit 2										DOCKET NUMBER (2) 0 5 0 0 0 2 3 7 1 OF 0 2										PAGE (3) 1 OF 0 2	
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TITLE (4) Unit 2 Reactor Scram											
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EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)											
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES			DOCKET NUMBER(S)								
0	6	1	9	8	5	8	5	0	2	8	0	0	0	7	1	8	8	5	N/A	0 5 0 0 0
										N/A			0 5 0 0 0							

OPERATING MODE (9) N		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)															
POWER LEVEL (10) 0 0 1	20.402(b)				20.406(c)				<input checked="" type="checkbox"/> 50.73(a)(2)(iv)				73.71(b)				
	20.406(a)(1)(i)				50.38(c)(1)				<input type="checkbox"/> 50.73(a)(2)(v)				73.71(c)				
	20.406(a)(1)(ii)				50.38(c)(2)				<input type="checkbox"/> 50.73(a)(2)(vi)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)				
	20.406(a)(1)(iii)				50.73(a)(2)(i)				<input type="checkbox"/> 50.73(a)(2)(viii)(A)								
	20.406(a)(1)(iv)				50.73(a)(2)(ii)				<input type="checkbox"/> 50.73(a)(2)(viii)(B)								
20.406(a)(1)(v)				50.73(a)(2)(iii)				<input type="checkbox"/> 50.73(a)(2)(ix)									

LICENSEE CONTACT FOR THIS LER (12)																	
NAME Ronald Jackson						(X-549)						TELEPHONE NUMBER 8 1 5 9 4 2 - 2 9 2 0					

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)											
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS	CAUSE	SYSTEM
X	A	A	I	I	V	I	2	0	8	N	

SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)										<input checked="" type="checkbox"/> NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

While performing a Unit 2 shutdown per procedure DGP 2-4, Unit 2/3 Shutdown from Power Operation to Hot Standby, with the unit in hot standby, the unit received an east scram discharge volume (SDV) high high water level alarm and a subsequent reactor scram. It was determined that the source of water causing the east SDV high high water level scram was due to excessive water leakage through the scram outlet valve, CV2-305-127, on control rod drive (CRD) J-6. Maintenance had just been completed on the scram outlet valve and CRD J-6 had been placed back in service approximately one hour before the reactor scram had occurred. Upon later investigation, the scram outlet valve was dismantled and a broken metal piece was found lodged under the valve seat causing the excessive leakage. The line was flushed to ensure no metal pieces remained and the scram outlet valve was repaired and reinstalled.

The safety significance of this event was minimal since the reactor protection and safety systems operated as designed. This is a first occurrence of this type.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1) Dresden Nuclear Power Station, Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 2 3 7							LER NUMBER (6)			PAGE (3)		
								YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
								8 5	—	0 2 8	—	0 0	0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

While performing a Unit 2 shutdown per procedure DGP 2-4, Unit 2/3 Shutdown from Power Operation to Hot Standby, with the unit in hot standby, the unit received an east scram discharge volume (SDV) high high water level alarm and a subsequent reactor scram. It was determined that the source of water causing the east SDV high high water level scram was due to excessive water leakage through the scram outlet valve, CV2-305-127, on control rod drive (CRD) J-6.

CRD J-6 had just been placed back in service approximately one hour before the reactor scram had occurred. Maintenance had been performed on the scram outlet valve and manual isolation valve 2-305-102. Maintenance work on the CRD included replacement of a badly worn seat on the scram outlet valve and complete replacement of manual isolation valve 2-305-102. Valve 305-102 had been replaced because of a broken stem and seat. After the reactor scram, the CRD was placed out-of-service and the scram outlet valve was dismantled. A broken metal piece was found lodged under the valve seat causing the excessive leakage. Upon investigation, it was determined that the metal piece was part of the original 2-305-102 valve stem that had broken. The line was flushed to ensure no metal pieces remained and the scram outlet valve was repaired and reinstalled. The CRD was placed back in service and scram tested with no further difficulties.

The safety significance of this event was minimal since the reactor protection and safety systems operated as designed. This is a first occurrence of this type.



Commonwealth Edison
Dresden Nuclear Power Station
R.R. #1
Morris, Illinois 60450
Telephone 815/942-2920

July 18, 1985

DJS Ltr #85-750

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Licensee Event Report #85-028-0, Docket #050237 is being submitted as required by Technical Specification 6.6, NUREG 1022 and 10 CFR 50.73 (a)(2)(iv).

J. A. Cusla
for

D.J. Scott
Station Manager

DJS/kjl

Enclosure

cc: J.G. Keppler, Regional Administrator, Region III
File/NRC
File/Numerical

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