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December 12, 1996  
6730-96-2369

U. S. Nuclear Regulatory Commission  
Attention: Document Control Desk  
Washington, DC 20555

Dear Sir:

SUBJECT: Oyster Creek Nuclear Generating Station  
Docket No. 50-219  
Monthly Operating Report - November, 1996

In accordance with the Oyster Creek Nuclear Generating Station Operating License No. DPR-16, Appendix A, Section 6.9.1, enclosed are two (2) copies of the Monthly Operating Data (Gray Book information) for the Oyster Creek Nuclear Generating Station.

If you should have any questions, please contact Ms. Brenda DeMerchant, Oyster Creek Regulatory Affairs Engineer, at 609-971-4642.

Very truly yours,

Michael B. Roche  
Vice President & Director  
Oyster Creek

MBR/BDeM/gl

Enclosures

cc: Administrator, Region I (2 copies)  
NRC Project Manager  
NRC Resident Inspector

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## SUMMARY

### November, 1996

Oyster Creek was off-line at the beginning of November. The plant was placed on-line at 0927 on 11/7/96 and reached full load at 1000 on 11/8/96. The plant remained at full load for the remainder of the month.

The plant generated 353,051 net MWH during the month and achieved an MDC net capacity factor of 79.2%.

**MONTHLY OPERATING REPORT**  
**Licensee Event Reports - November, 1996**

LER 96-008, filed November 25, 1996:

On October 25, 1996 at approximately 1159 hours a manual scram of the reactor was initiated in accordance with plant procedures due to a confirmed Main Generator Runback. Plant response to the generator runback and manual scram was normal. All control rods inserted fully, and plant response was as designed. Initial troubleshooting did not identify the initiating component. However, subsequent monitoring identified an invalid temperature switch actuation, which were then disconnected pending further analysis. Corrective actions include continued fault analysis on the existing temperature sensing switches. New temperature sensing devices will be evaluated and tested prior to returning the temperature portion of the main generator runback circuit to service.

LER 96-009, filed November 25, 1996:

On October 27, 1996 at 2358 hours, power to the 4160 VAC 1D bus was lost due to a ground fault on the cable between Diesel Generator 2 and the 1D bus. The plant was in a cold shutdown condition for a maintenance outage prior to this event.

The loss of the bus led to: 1) a trip of Reactor Protection System Channel 2; and 2) a full reactor scram signal and main steam line isolation, which isolated the reactor vent path being used, as designed.

Immediate actions were taken to restore the actuated systems, and reestablish the reactor vent path. Additional corrective actions were taken to replace the faulted cable and test the other diesel cables. Long term corrective actions include sending the failed cable to an outside testing laboratory for fault analysis.

LER 96-010, filed November 26, 1996:

On October 16, 1996, while performing a routine refueling outage surveillance on the Appendix R Remote Shutdown Panel (RSP) during the 16R Refueling Outage, one valve of the Isolation Condenser System did not operate as expected. This valve is designed to achieve an open position when an RSP relay is activated. The apparent cause of the occurrence was dirty contacts on the control relay for this valve in the RSP. The contacts on the affected relay were cleaned and the valve circuit was retested satisfactorily. The safety significance of this occurrence is considered minimal because other systems were available to remove decay heat from the reactor and keep the core covered during an Appendix R fire in the Control Room. Engineering is evaluating the problem with the contacts on the relays used in the Remote Shutdown System to determine the appropriate action to take to prevent recurrence.

On October 26, 1996, a pressure drop test indicated primary containment leakage was above the Technical Specification limit. A Torus to Drywell vacuum breaker valve cover was found to be leaking. The valve cover was repaired and gross containment leakage was calculated to be below technical specification allowable leakage. Containment integrity was required for a five day period prior to discovery.

The cause of this event was determined to be less than adequate self-checking which resulted in failure to correctly reassemble V-26-005 valve cover during the 16R outage. Procedure changes will require emphasis on self-checking to ensure that the cover plate is level during and after the torquing sequence. An evaluation of the methodology used in performing Local Leak Rate Testing (LLRT) of the torus/drywell vacuum breakers will be done.

Additionally, this specific human performance error, as well as others, will be discussed at weekly craft/management interface meetings now in progress

AVERAGE DAILY POWER LEVEL  
NET MWe

DOCKET # . . . . .	50-219
UNIT . . . . .	Oyster Creek #1
REPORT DATE . . . . .	12/2/96
COMPILED BY. . . . .	Paul G. Edelmann
TELEPHONE #. . . . .	(609) 971- 4097

Month: November, 1996

DAY	MW	DAY	MW
1	0	16	639
2	0	17	639
3	0	18	640
4	0	19	638
5	0	20	638
6	0	21	640
7	134	22	640
8	580	23	641
9	630	24	639
10	633	25	641
11	634	26	640
12	637	27	639
13	637	28	641
14	636	29	641
15	638	30	641

Oyster Creek Station #1  
Docket No. 50-219

Refueling Information - November, 1996

Name of Facility: Oyster Creek Station #1

Scheduled date for next refueling shutdown: September 1998

Scheduled date for restart following refueling: October 1998

Will refueling or resumption of operation thereafter require a  
Technical Specification change or other license amendment? No

Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

1. General Electric Fuel Assemblies - Fuel design and performance analysis methods have been approved by the NRC.

The Number of fuel assemblies:	(a)	in the core	=	560
	(b)	in the spent fuel storage pool	=	2236
	(c)	in the new fuel storage vault	=	8

The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

Present Licensed Capacity: 2645

The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

Full core discharge capacity to the spent fuel pool was lost after the 1996 refueling outage.

# OPERATING DATA REPORT

## OPERATING STATUS

1. Docket: 50-219
2. Reporting Period: November, 1996
3. Utility Contact Paul G. Edelmann (609)-971-4097
4. Licensed Thermal Power (MWt): 1930
5. Nameplate Rating (Gross MWe):  $687.5 \times 0.8 = 550$
6. Design Electrical Rating (Net MWe) 650
7. Maximum Dependable Capacity (Gross MWe): 641
8. Maximum Dependable Capacity (Net MWe): 619
9. If Changes Occur Above Since Last Report, Give Reasons: None
10. Power Level to Which Restricted, If Any (Net MWe): None
11. Reason For Restriction, If Any: None

	<u>Month</u>	<u>Year</u>	<u>Cumulative</u>
12. Report Period Hours	720.0	8040.0	236160.0
13. Hours RX Critical	595.1	6511.9	160844.8
14. RX Reserve Shutdown Hours	0.0	0.0	918.2
15. Hours Generator On-Line	566.6	6363.1	157173.8
16. UT Reserve Shutdown Hours	0.0	0.0	0.0
17. Gross Thermal Energy (MWH)	1075438	11960758	272480251
18. Gross Electric Energy (MWH)	367432	4017711	91435286
19. Net Electric Energy (MWH)	353051	3862906	87732071
20. UT Service Factor	10.1	79.1	66.6
21. UT Available Factor	78.7	79.1	66.6
22. UT Capacity Factor (MDC Net)	79.2	77.6	60.6
23. UT Capacity Factor (DER Net)	75.4	73.9	57.2
24. UT Forced Outage Rate	21.3	8.6	9.8
25. Forced Outage Hours	153.5	599.2	17137.4
26. Shutdowns Scheduled Over Next 6 Months (Type, Date, Duration)	None		
27. Currently Shutdown, Estimated Startup Date:	N/A		

## UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO: 50-219  
 UNIT NAME: Oyster Creek  
 DATE: December 5, 1996  
 COMPLT'D BY: David M. Egan  
 TELEPHONE: 609/971-4818

REPORT MONTH: November 1996

No.	DATE	TYPE F: Forced S: Scheduled	DURATION (hours)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER (2)	CORRECTIVE ACTIONS/COMMENTS
7	10/25/96	F	153.4	a	2	Manual Scram at 1200 on 10/25/96 in response to a Generator Runback. Plant on-line on 11/7/96 at 0927.

### SUMMARY:

#### (1) REASON

- |                                |                                 |
|--------------------------------|---------------------------------|
| a. Equipment Failure (Explain) | e. Operator Training & Lic Exam |
| b. Maintenance or Test         | f. Administrative               |
| c. Refueling                   | g. Operational Error (Explain)  |
| d. Regulatory Restriction      | h. Other (Explain)              |

#### (2) METHOD

1. Manual
2. Manual Scram
3. Automatic Scram
4. Other (Explain)