

LICENSEE EVENT REPORT

CONTROL BLOCK: _____ (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

| | | | | | | | | | | | | | | | | | | | | | | | | | | | |
|----|---|---------------|---|---|---|---|----|----|-----------------|---|---|---|---|---|---|---|---|---|----|----|------------------|---|---|---|---|----|----|
| 01 | V | A | S | P | S | 2 | 2 | 0 | 0 | - | 0 | 0 | 0 | 0 | 0 | 0 | 0 | 3 | 4 | 1 | 1 | 1 | 1 | 4 | 5 | | |
| 8 | 9 | LICENSEE CODE | | | | | 14 | 15 | LICENSEE NUMBER | | | | | | | | | | 25 | 26 | LICENSEE TYPE JO | | | | | 57 | 58 |

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| 01 | L | 6 | 0 | 5 | 0 | 0 | 0 | 2 | 8 | 0 | 7 | 0 | 8 | 0 | 3 | 8 | 3 | 8 | 0 | 9 | 0 | 2 | 8 | 3 | 9 |
| 7 | 8 | 60 | 61 | DOCKET NUMBER | | | | | 68 | 69 | EVENT DATE | | | | | 74 | 75 | REPORT DATE | | | | | 80 | | |

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 | Type "C" testing performed during the 1983 refueling outage for Unit No. 2

03 | disclosed an "as found" leakage exceeding the allowable leakage rate delineated in

04 | T.S.-4.4.C. This is contrary to T.S.-3.8.A.1 and is reportable per T.S.6.6.2.b.(2).

05 | Actual containment leak rate would have been considerably less than the value

06 | leak rate computed for the test. Therefore, the health and safety of the public

07 | would not have been affected.

08 | _____

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|----------------------|---|---------------|----|-----------------|----|-----------------|----|----------------|----|----------------------|----|------------------|---------------|----------------------|---------------|------------------------|-----------------------|----|-----------------|----|-------------|----|--------------|----|----|
| 09 | S | D | 11 | E | 12 | B | 13 | V | A | L | V | E | X | 14 | X | 15 | X | 16 | | | | | | | |
| 7 | 8 | 9 | 10 | 11 | 12 | 13 | 14 | 15 | 16 | 17 | 18 | 19 | 20 | 21 | 22 | 23 | 24 | 25 | 26 | 27 | 28 | 29 | 30 | 31 | 32 |
| LER/RO REPORT NUMBER | | EVENT YEAR | | CAUSE CODE | | CAUSE SUBCODE | | COMPONENT CODE | | | | | COMP. SUBCODE | | VALVE SUBCODE | | SEQUENTIAL REPORT NO. | | OCCURRENCE CODE | | REPORT TYPE | | REVISION NO. | | |
| 17 | | 83 | | E | | B | | VALVEX | | | | | X | | X | | 033 | | 03 | | L | | 0 | | |
| ACTION TAKEN | | FUTURE ACTION | | EFFECT ON PLANT | | SHUTDOWN METHOD | | HOURS | | ATTACHMENT SUBMITTED | | NFRD-4 FORM SUB. | | PRIME COMP. SUPPLIER | | COMPONENT MANUFACTURER | | | | | | | | | |
| X | | X | | Z | | Z | | 0000 | | Y | | N | | A | | X999 | | | | | | | | | |

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 | The major cause of leakage was determined to be deterioration and wear of the

11 | valves due to normal usage. The valves have been repaired and retested with

12 | satisfactory results.

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|-------------------------------|---|---------------------|----|--------------------|----|---------------------|-----|-----------------------|----|------------------|----|
| 15 | H | 28 | 0 | 0 | 0 | 29 | N/A | B | 31 | Type "C" testing | 32 |
| 7 | 8 | 9 | 10 | 11 | 12 | 13 | 14 | 15 | 16 | 17 | 18 |
| FACILITY STATUS | | % POWER | | OTHER STATUS | | METHOD OF DISCOVERY | | DISCOVERY DESCRIPTION | | | |
| 15 | | 000 | | N/A | | B | | Type "C" testing | | | |
| ACTIVITY CONTENT | | RELEASED OF RELEASE | | AMOUNT OF ACTIVITY | | LOCATION OF RELEASE | | | | | |
| 16 | | Z | | N/A | | N/A | | | | | |
| PERSONNEL EXPOSURES | | NUMBER | | TYPE | | DESCRIPTION | | | | | |
| 17 | | 000 | | Z | | N/A | | | | | |
| PERSONNEL INJURIES | | NUMBER | | TYPE | | DESCRIPTION | | | | | |
| 18 | | 000 | | | | N/A | | | | | |
| LOSS OF OR DAMAGE TO FACILITY | | TYPE | | DESCRIPTION | | | | | | | |
| 19 | | Z | | N/A | | | | | | | |
| PUBLCITY ISSUED | | DESCRIPTION | | | | | | | | | |
| 20 | | N | | N/A | | | | | | | |

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PDR ADOCK 05000281
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ATTACHMENT 1
SURRY POWER STATION, UNIT NO. 2
DOCKET NO: 50-281
REPORT NO: 83-033/03L-0
EVENT DATE: 08-03-83

TITLE OF THE EVENT: CONTAINMENT ISOLATION VALVES LEAKAGE

1. Description of the Event

With the Unit in a Refueling outage, it was discovered through performance of Periodic Test 16.4 (Containment Isolation Valve Leakage, Type 'C' testing) that the "As Found" leakage exceeded the allowable leakage specified in Technical Specification 4.4.C. This is contrary to Tech. Spec. 3.8.A.1 and is reportable in accordance with Tech. Spec. 6.6.2.b.(2).

2. Probable Consequences and Status of Redundant Equipment

Total leakage for the test is computed by summing the leak rates of all the valves tested. This very conservative method does not take into consideration two or more isolation valves in series for the same containment penetration. Actual leakage through the penetration would be that of the valve with the smallest leak rate. Since the actual containment leak rate would have been considerably less than the valve leak rate computed by the test, the health and safety of the public would not have been affected.

3. Cause

The major cause of leakage was determined to be deterioration and wear of the valves due to normal usage.

4. Immediate Corrective Action

As each valve with excessive leakage was identified, maintenance activities were initiated to have the valve repaired.

5. Subsequent Corrective Action

The affected valves have or will be repaired and subsequent testing will verify that the total combined leakage for the valves is within specification.

6. Action Taken to Prevent Recurrence

The containment leakage will continue to be monitored as required by 10CFR 50, Appendix J. In addition, a Type A test will be performed to verify containment integrity. An engineering review of the testing data and equipment selection will be conducted to determine possible corrective actions/modifications.

7. Generic Implications

None.

Vepco

VIRGINIA ELECTRIC AND POWER COMPANY
Surry Power Station
P. O. Box 315
Surry, Virginia 23883

03 SEP 6 A 9:34

SEP 2 1983

Serial No: 83-061

Docket No: 50-281

License No: DPR-37

Mr. James P. O'Reilly
Regional Administrator
Suite 2900
101 Marietta Street, NW
Atlanta, Georgia 30303

Dear Mr. O'Reilly

Pursuant to Surry Power Station Technical Specifications, the Virginia Electric and Power Company hereby submits the following Licensee Event Report for Surry Unit 2.

Report Number

83-033/03L-0

Applicable Technical Specification

T. S. 6.6.2.b(2)

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be reviewed by Safety Evaluation and Control.

Very truly yours,

J. L. Wilson
J. L. Wilson
Station Manager

Enclosure

cc: Document Control Desk, USNRC
016 Phillips Bldg.
Washington, D. C. 20555

DESIGNATED ORIGINAL
Certified By Barline Scott

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