

LICENSEE EVENT REPORT

EXHIBIT A

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Arkansas Nuclear One, Unit One										DOCKET NUMBER (2) PAGE (3)									
TITLE (4) Anticipatory Reactor Trip Following Feedwater Pump Trip										0 5 0 0 0 3 1 1 3 1 0 F 0 1									
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)									
Month	Day	Year	Year	Sequential Number	Revision Number	Month	Day	Year	Facility Names					Docket Number(s)					
01	06	01	28	01	01	01	07	01	28						0 5 0 0 0 1				
OPERATING MODE (9) N										THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §:									
(Check one or more of the following) (11)																			
POWER LEVEL (10) 0 4 5		20.402(b)		20.405(c)		X		50.73(a)(2)(iv)		73.71(b)									
		20.405(a)(1)(i)		50.36(c)(1)				50.73(a)(2)(v)		73.71(c)									
		20.405(a)(1)(ii)		50.36(c)(2)				50.73(a)(2)(vii)		Other (Specify in		Abstract below and in Text, NRC Form 366A)							
		20.405(a)(1)(iii)		50.73(a)(2)(i)				50.73(a)(2)(viii)(A)											
		20.405(a)(1)(iv)		50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)											
		20.405(a)(1)(v)		50.73(a)(2)(iii)				50.73(a)(2)(x)											
Name										LICENSEE CONTACT FOR THIS LER (12)									
Patrick C. Rogers, Plant Licensing Engineer										Telephone Number									
										Area									
										Code									
										5 0 1 1 9 6 4 - 1 3 1 1 0 1 0									
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																			
Cause	System	Component	Manufacturer	Reportable to NPRDS	Cause	System	Component	Manufacturer	Reportable to NPRDS										
SUPPLEMENT REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)									
Yes (If yes, complete Expected Submission Date) No										Month Day Year									
ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)																			

On 6/2/85, at 1959 hours, an anticipatory reactor trip occurred on a loss of both Main Feedwater (MFW) pumps signal while operating at ~45% power. MFW pump P-1A was operating in automatic and MFW pump P-1B was being returned to service following repairs. The discharge pressures of the pumps had been equalized, however, the recirculation valve for P-1B was open resulting in P-1B flow returning to the condenser hotwell and P-1A flow supplying both steam generators. Not realizing the P-1B recirculation valve was open, operations closed the MFW trains cross-connect valve. P-1A tripped on high discharge pressure, P-1B continued to operate, and an anticipatory reactor trip occurred from a loss of both MFW pumps signal. Investigation revealed that the P-1B Reactor Protection System (RPS) trip bistable had not been reset when the pump was previously tripped for repair work. Emergency feedwater also actuated from the loss of both MFW pumps signal and maintained steam generator water level until the auxiliary feedwater system was placed in service. Also, the hydraulic speed control feedback system was improperly set for both MFW pumps which prevented P-1A from promptly responding and running back when the cross-connect valve was closed. The hydraulic speed control system was adjusted for both pumps. Operating procedures have been revised to ensure RPS trips have been reset prior to placing a pump in service and that recirculation valves are closed prior to closing the cross-connect valve.

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July 2, 1985

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U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: Arkansas Nuclear One - Unit 1
Docket No. 50-368
License No. NPF-6
Licensee Event Report
No. 85-005-00

Gentlemen:

In accordance with 10CFR50.73(a)(2)(iv), attached is the subject report concerning a spurious anticipatory reactor trip which occurred due to a signal indicating loss of both main feedwater pumps.

Very truly yours,

J. Ted Enos
Manager, Licensing

JTE:RJS:ds

Attachment

cc: Mr. Richard C. DeYoung
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, DC 20555

Mr. Norman M. Haller, Director
Office of Management & Program Analysis
U. S. Nuclear Regulatory Commission
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