



231 W. Michigan, P.O. Box 2046, Milwaukee, WI 53201-2046

(414) 221-2345

VPNPD-96-075

10 CFR 50.4

10 CFR 50.90

September 26, 1996

Document Control Desk
US NUCLEAR REGULATORY COMMISSION
Mail Station P1-137
Washington, DC 20555

Gentlemen:

DOCKETS 50-266 AND 50-301
SUPPLEMENT TO TECHNICAL SPECIFICATIONS
CHANGE REQUESTS 188 AND 189
POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2

By letters dated June 4, 1996, Wisconsin Electric Power Company, Licensee for the Point Beach Nuclear Plant, submitted Technical Specifications Change Requests 188 and 189. These requests proposed amendments to the Point Beach Technical Specifications that were identified by analyses performed in support of Unit 2 operations following replacement of steam generators this fall. The revisions proposed in Change Request 188 modify Technical Specification Section 15.2.3, "Limiting Safety System Settings and Protective Instrumentation," and Section 15.5.3, "Design Features - Reactor." The revisions proposed in Change Request 189 modify Sections 15.2.1, "Safety Limit, Reactor Core;" 15.2.3, "Limiting Safety System Settings, Protective Instrumentation;" and 15.3.1.G, "Operational Limitations."

We are providing additional information as attachments to this letter. This additional information includes the following:

Setpoint uncertainty analysis for the proposed Technical Specification setting for Steam Generator low-low level which shows that appropriate margins between the safety analysis acceptance criteria and Technical Specification limits are maintained.

An error in the steam generator tube rupture dose analysis results was discovered and has been corrected in the FSAR mark-up for the steam generator tube rupture accident.

An additional Technical Specifications basis is being changed in Section 15.3.4. This basis contains a value for secondary side water volume in a steam generator for the Westinghouse Model 44 steam generator which will no longer be used at PBNP when the Unit 2 steam generators are replaced.

Clarification of the FSAR mark-up that showed the total primary heat output changing from 1518.5 MWt to 1524.5 MWt.

010054

NRC PDR:
change distribution
subsidiary of Wisconsin Energy Corporation

1 1 NRC
APD 1/1

9610020093 960926
PDR ADDOCK 05000266
PDR

Document Control Desk

September 26, 1996

Page 2

The previously provided uncertainty analysis related to the Overtemperature and Overpower-delta-T setpoints contained information which is proprietary to the Westinghouse Electric Corporation. Accordingly, we requested that this information be withheld from public disclosure. We are providing a non-proprietary version of this material together with an affidavit in accordance with the requirements of 10 CFR 2.790.

We have determined that the proposed amendments do not involve a significant hazards consideration, authorize a significant change in the types or total amounts of any effluent release, or result in any significant increase in individual or cumulative occupational exposure. Therefore, we conclude that the proposed amendments meet the requirements of 10 CFR 51.22(c)(9) and that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared. The original "No Significant Hazards" determinations for operation under the proposed Technical Specifications remain applicable.

Our replacement outage is presently scheduled to commence on October 5, 1996. We request approval of these changes by October 31, 1996, with implementation to occur prior to startup of Unit 2 from the outage. This will ensure the amendments are implemented and appropriate training is performed prior to operation.

If you require additional information, please contact us.

Sincerely,




Bob Link
Vice President
Nuclear Power

CAC

cc: NRC Resident Inspector
NRC Regional Administrator
PSCW

Subscribed to and sworn before me on
this 26th day of September, 1996.



Notary Public, State of Wisconsin

My commission expires 10/27/94