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the southern electric system

February 12, 1993

ELV-05217 003035

Docket Nos. 50-424

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M82725

U. S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, D. C. 20555

VOGTLE ELECTRIC GENERATING PLANT ADDITIONAL INFORMATION TO SUPPORT POWER UPRAYING

Gentlemen:

Georgia Power Company letter ELV-03375 dated February 28, 1992, requested a revision to the operating license for Vogtle Electric Generating Plant (VEGP) Units 1 and 2 that would allow operation at an uprated power level. This letter presents additional information supporting the requested license change and the analysis in ELV-03375.

The additional information concerning the effects of operating at uprated power level on steam generator tubes (SGT) is contained in attachment 1. The change in the SGT minimum acceptable wall thickness from 0.014 inch to 0.016 inch was a result of operating with a reduced steam pressure. The calculation of the revised minimum wall thickness used the same methodology as was previously used. Operation at the uprated condition will result in a slight increase in the number of steam generator tubes that may be susceptible to antivibration bar (AVB) wear (5-10 tubes over the life of the plant). However, no significant increase in the rate of AVB wear is projected. Attachment 4 contains the results of steam generator tube inspections for VEGP.

The additional information concerning the calculation of peak containment pressure is included as attachment 2. The peak containment pressure currently in the Final Safety Analysis Report was calculated at a power level equivalent to the uprated power level reported in ELV-03375. The reductions in the calculated peak containment pressures were due to the use of revised methods for calculating the mass and energy releases to the containment, more accurate accounting of heat sinks, and taking credit for the fan coolers required to be operable according to the current Technical Specifications. The methodology for calculating the mass and energy release and the containment pressures has been previously used for plants of similar design.

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Georgia Power

U. S. Nuclear Regulatory Commission

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The revised calculation of the turbine trip event with conservative assumptions to evaluate the overpressure protection design is described in section 15.2.3.3. of appendix A to enclosure 4 of letter ELV-02166 dated November 29, 1990. Additional information concerning the overpressure protection evaluation is contained in attachment 3. The relief capacity of the safety valves has been evaluated to demonstrate that it is sufficient for the design basis event if reactor scram is initiated on the second safety grade signal from the reactor protection system.

The original licensing basis, the Vantage 5 licensing submittal, and the uprating license submittal determined values for the overpower delta T(OPDT) and overtemperture delta T(OTDT) setpoints using the methodology described in WCAP-8745-P-A. The setpoints were calculated to support the uprated power level in conjunction with the transition to Vantage 5 fuel as documented in ELV-02166. Subsequent to the calculation of the OPDT and OTDT setpoints in support of the Vantage 5 licensing submittal, a more conservative approach was used in the generation of the FAI function of the OTDT setpoint. This resulted in the revised values for the OTDT setpoint presented in the uprating licensing submittal (ELV-03375).

The evaluations contained in our letter ELV-03375 included the effects of operating at a reduced temperature (T hot reduction). The low temperature overpressure protection system and its associated setpoints will not be affected by T hot reduction. The evaluations and analyses performed in support of the application for this license amendment, and presented in our letters ELV-03375 and ELV-02166 including the inadvertent opening of a steam generator relief or safety valve and steam system piping failure utilized methods that have previously been accepted by the NRC for use on the Vogtle Electric Generating Plant (VEGP) or accepted for use by the NRC since the initial licensing of VEGP.

The maximum sump temperature at the time of switchover to recirculation was calculated to be 244°F, which is about 8°F different from the current licensing basis calculation. This value is recorded on page 3-200 of enclosure 5 of letter ELV-03375. This number was used in the reverification of piping stress analyses. Based on this value, it was concluded that the piping stresses will remain within their original acceptance limits.

The above information and the additional information in attachments 1, 2, 3, and 4 is supplemental to the information previously transmitted to the NRC and does not represent any changes to the previously submitted material.

Sincerely,

C.K. McCoy

CKM/HWM/clr xc: (See next page) U. S. Nuclear Regulatory Commission

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Attachments:

- Steam Generator Tube Effects of Rerating
 VEGP Containment Analyses
- 3. Pressurizer Safety Valve Sizing
- 4. Steam Generator Tube Inspection Results

xc: Georgia Power Company

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ELV-05217 003035

ATTACHMENT 1 STEAM GENERATOR TUBE EFFECTS OF RERATING

- 1. The minimum acceptable tube wall thickness for VEGP steam generator tubes for the rerated conditions was determined to be 0.016 inch using Regulatory Guide 1.121 guidelines. This compares to a minimum acceptable wall thickness of 0.014 inch for the current Vogtle design conditions. This increase in the minimum acceptable tube wall thickness is due to the decreased steam pressure and resultant increased differential pressure across the tubes for the rerated conditions. The same Regulatory Guide 1.121 methodology was used for both calculations.
- 2. Wear of the steam generator (SG) tubes at the antivibration bar (AVB) locations is principally dependent on the boundary conditions of the tube (i.e., whether the AVBs provide adequate support of tubes), which is a statistical issue for the population of tubes affected by the AVBs. Operating experience has shown that a wide range of wear rates may exist among the tubes, and that only a small percentage of tubes will wear sufficiently to require plugging over the operating life of the SG. Although changes in operating conditions may affect the wear rate (change in through wall depth (TWD) on individual tubes), this effect is negligible compared to the variability in tube boundary conditions indicated by operating experience.

Current projections for AVB wear are that no more than 0.6 percent of the tubes in the model F SG may require plugging over the operating life of the tubes. The operating experience of plants (including those which have uprated) with model F SGs is that plugging levels are below the plugging projection. The only exception is a foreign plant in which it was identified that the AVB/U-bend assembly was improperly assembled. Uprating is estimated to increase the number of tubes requiring plugging due to AVB wear by 10-16 percent; therefore, the upper limit of projected plugging for uprated conditions would be less than 0.7 percent of the tubes.

For the operating model F SGs, in no case has progression of wear at the AVB intersections resulted in tube leakage. An individual tube has experienced a growth rate of approximately 40 percent between inspections over a 3-year time interval in a plant which was previously uprated. For the same plant, the average tube wear growth rate at the AVBs was 3.8 percent/year TWD over a 5-year interval. The structural limit (based on ASME Code criteria) for AVB wear is approximately 75 percent TWD. It is not expected that uprating will have significant effect on the wear growth rate on individual tubes or result in increased potential for through wall wear or primary to secondary leakage at the AVB intersections.

ATTACHMENT 2 VEGP CONTAINMENT ANALYSES LOSS OF COOLANT ACCIDENT (LOCA) Double Ended Hot Leg Guillotine (DEHLG) The current licensing basis FSAR double ended hot leg guillotine break case resulted in a calculated containment peak pressure of 38.24 psig occurring at 18.3 seconds. The Vogtle rerating analysis provided a containment peak pressure of 36.54 psig occurring at 17.3 seconds. The essential differences between the two analyses are as follows: 1. The mass and energy release evaluation model used in performing the FSAR analysis of record is identified and documented in WCAP-8264-P-A. The evaluation model used in performing the VEGP rerate analyses is documented in WCAP-10325-P-A. Reactor coolant system temperature uncertainty 6°F versus 4°F (FSAR) RCS pressure 2250 + 50 psi versus 2250 + 30 psi (FSAR) b) 2. Revised containment structural heat sinks were modeled in completing the rerate analyses. These revised heat sinks were based upon a more accurate but conservative accounting which better reflects the current VEGP design as compared to the data available at the time when the FSAR analyses were being performed. a) Initial containment pressure assumption was 3.0 psig vs 0.3 psig (FSAR) The duration of the transient (i.e., less than 30 seconds), the mass and energy release and amount of containment structural heat sinks strongly govern the calculated pressure profile. Double ended Pump Suction Guillotine (DEPSG) The current licensing basis FSAR double ended pump suction guillotine break case resulted in a calculated containment peak pressure of 35.62 psig occurring at 1108 seconds. The VEGP rerating analysis provided a containment peak pressure of 34.61 psig occurring at 17.6 seconds. The essential differences between the two analyses are as follows: The mass and energy release evaluation model used in performing the FSAR analysis of record is identified and documented in WCAP-8264-P-A. The evaluation model used in performing the VEGP rerate analyses is documented in WCAP-10325-P-A. a) Reactor coolant system temperature uncertainty 60F versus 40F RCS pressure 2250 + 50 psi versus 2250 + 30 psi (FSAR) Steam/water mixing model C) d) 1979 AMS decay heat model

ATTACHMENT 2 (CONTINUED)

VEGP CONTAINMENT ANALYSES

- 2. Revised containment structural heat sinks were modeled in completing the rerate analyses. These revised heat sinks were based upon a more accurate but conservative accounting which better reflects the current VEGP plant design as compared to the data available at the time when the FSAR analyses were being performed.
 - a) Initial containment pressure assumption was 3.0 psig vs. 0.3 psig (FSAR)
- 3. Increased number of fan coolers available. The current FSAR analyses assumed two. The rerate analysis assumed four fan coolers available.

Note that the calculated pressure profile has two peaks. The maximum calculated pressure for the rerated condition occurred at the fir peak; it occurred at the second peak for the current licensing table case.

Main Steam Line Break (MSLB) Containment Integrity

The current licensing basis limiting FSAR case for peak pressure was the 0.14 ft² split at hot shutdown case which resulted in a calculated containment peak pressure of 41.9 psig occurring at 1784 seconds. The VEGP rerating limiting case was the 0.22 ft² double ended rupture without entrainment at 30 percent power. This case produced a peak containment pressure of 32.7 psig occurring at 2177 seconds. The essential differences between the two analyses are as follows:

Containment Model Assumptions:

- Revised containment structural heat sinks were modeled in completing the rerate analyses. These revised heat sinks were based upon a more accurate but conservative accounting which better reflects the current Vogtle plant design as compared to the data available at the time when the FSAR analyses were being performed.
- Credit was taken for the operation of 4 of the 8 containment coolers since this is the number of coolers required to be operable by the Technical Specifications.
- 3. Initial containment pressure assumed was 3.0 psig vs. 0.3 psig (FSAR)

MSLB Mass and Energy Release Assumptions:

- 1. Main feedwater flow rate The values used in the rerate analysis are in general greater than what was used in the FSAR analysis.
- Auxiliary feedwater enthalpy The current FSAR analysis used 429.5 Btu/lbm and the rerate analysis assumed 101 Btu/lbm.

ATTACHMENT 2 (CONTINUED)

YEGP CONTAINMENT ANALYSES

3. Decay Heat Model - The FSAR analysis used the 1971 model whereas the rerate analysis used the 1979 model.

In general, the containment model used to calculate the containment response following a MSLB inside containment was identical to that used for the LOCA containment integrity analyses. The lower calculated containment peak pressures for the rerate analysis were directly influenced by the increased number of containment fan coolers available (2 to 4) and improved structural heat sink modeling with respect to containment modeling.

ATTACHMENT 3

PRESSURIZER SAFETY VALVE SIZING

It is required that the pressurizer safety valves be designed with sufficient capacity to prevent the RCS pressure from exceeding 110 percent of design pressure following the worst reactor coolant system pressure transient. Justification for the overpressure protection (safety valve sizing) for Westinghouse PWRs is given by the generic analyses presented in WCAP-7769. These generic analyses addressed the full power load rejection resulting from a turbine trip with concurrent loss of main feedwater. The analyses assumed no credit is taken for operation of reactor coolant system relief valves, steam line relief valves, steam dump system, pressurizer level control system, pressurizer spray, or direct reactor trip on turbine trip. Reactor trip is initiated by the second safety-grade signal from the reactor protection system. The generic analyses were performed assuming a core power of 3411 MWt with a 2-percent power uncertainty.

With the recent VEGP uprating submittal, the NRC has questioned the applicability of WCAP-7769 due to the increased power level. Vogtle Electric Generating Plant specific analyses have been performed which demonstrate that the pressurizer safety valves are sufficiently sized at the uprated power (102 percent of 3579 MWt). The analyses demonstrated that when crediting the high pressurizer pressure reactor trip for the loss of load/turbine trip event, 73 percent of the total capacity of the pressurizer safety valves was required. For subsequent reactor trips (i.e., overtemperature ΔT , high pressurizer water level, and low-low steam generator water level), 81 percent of the total capacity of the pressurizer safety valves was required. With respect to the steam generator safety valves, the flow out of the valves did not exceed nominal plant steam flow at any time during the analyzed transients.

ATTACHMENT 4

STEAM GENERATOR TUBE
INSPECTION RESULTS

**	28	13 5C+17.7	7 30	27	35
**	24 14	14 TSH+8.7	B NI	15	NI
** 28	28 18	18 2H+1.42	NI	13	NI
** 45 45	45 22 22	AV2 AV5	NDD NDD	14 17	17 25
47	47 24 24 24	24 AV2 AV4 AV5	15 15 NF	21 21 13	24 27 18
** 47	47 26	26 AV2	NI	10	INR
52	52 33 33		NI NI		
54 54 54	35 35 35 35 35 35	AV2 AV3 AV4 AV5	PV I	NF 21	13
40	40	44 4H+18.90	16	INR	NI
41	41 44 44	44 AV2 AV3	NI NI	NF 18	15 21
42		6H+13.00	26	INR I	DDM
	44		NDD	15	19
* 5 57	44	AV5	NDD	17	23

***** GROWTH RATE

* UNIT 1 STEAM GENERATOR NO. 1 *

ROW CLMN LOCATION 1R1 1R2 1R3 1R4 1R5 1R6 1R7 1R8 1R9 1R10 GRTHRTE 经财政 医乳状的 化环状异形式合物 解放性 医软件 医红色 数形式 物效的 医条件 经收款 医乳腺 经股份 经股份股份

WER	****	******	***	***	W 61 20	1
57	44	AV6	NDD	11	11	
**	57	45				
57	45	AV3	NI	11	15	
57	45	AV3 AV5	NI	12	INR	
**	40	47				
40	47	AV3	NDD	13	15	
40	47	AV4	NDD	13	18	
**	34	49				
34	49	BPC+6.96	25	29	INR	
**	13	52				
13	52	3H+19.54	18	17	NI	
**	35	52				
35	52	BPC+10.30	22	22	MBM	
35	52	BPC+10.30 IC+31.90	MBM	23	MBM	
**	55	55				
55	55	AV2	NDD	20	29	
55	55	AV3	NDD	11	22	
55	55	AV2 AV3 AV4	NDD NDD	18	29	
**	35	59 BPC+18.20				
35	59	BPC+18.20	11	INR	NI	
**	59	67				
59	67	7H+4.35	NI	UDS	NI	
**	55	73 6H+32.48				
55	73	6H+32.48	NI	UDS	NI	
**	46	87				
46	87	AV1+20.93	NI	UDS	NI	
**	39	92				
39	92	3H+1.03	NI	34	UDS	
**	42 1	04				
	104		NF	13	INR	
25	25 1	10	817	0.0	MPN	
20	110	AVD	NI	22	MBM	

GROWTH CATE

* UNIT 1 STEAM GENERATOR NO. 1 *

ROW CLMN LOCATION 1R1 1R2 1R3 1R4 1R5 1R6 1R7 1R8 1R9 1R10 GRTHRTE **BEZ QUIL MINISTER DE BUR BUR DES DES DES DES BUR DES DES BUR DES BURS DES BURS DE BUR**

** 26 116 26 116 AV6 NI 16 14

** 24 117

24 117 1C+15.31 NI UDS MBM 24 117 2C+14.17 NI 26 INF

* GROWTH RATE * UNIT 1 STEAM GENERATOR NO. 2 *

**	33 12	12 2H+16.58	NI	UDS	NI
		19 7C+0.00			
** 25	25 20	20 AV5	NI	20	19
** 17	17 27	27 7C+0.00	NI	NI	UDS
** 17 17	17 29 29	29 7C+1.32 7C-0.06	NI NI	UDS UDS	NI NI
14	33	33 5H+14.05	NI	21	МВМ
** 41 41	41 33 33	33 AV4 AV5	NI	15 17	20 19
** 32	32 39	39 1C+0.00	NI	UDS	NI
5.9	41	41 1C+5.21	NI	UDS	NI
20	44	44 3H+34.71			
	05 45	45 4C+6.11	NI	UDS	NI
** 25		45 1C+5.73	NI	UDS	NI
** 25		46 1C+5.44	NI	UDS	NI
46 46 46	50 50 50 50 50	AV1 AV2 AV4 AV5	NI NI NI NI	13 14 14	21 17

**** * GROWTH RATE * UNIT 1 STEAM GENERATOR NO. 2 *

** 45 56 45 56 AV2			
40 00 AVZ	N)	26	19
** 32 60 32 60 5C+6.26	NI	16	MBM
** 08 61 08 61 4H+4.36	NI	UDS	NI
** 27 62 27 62 6H+21.37	NI	21	MBM
** 55 64 55 64 AV5	NI	NDD	16
** 49 69 49 69 2C+17.73	NI	37	36
** 53 83	NI	19 12	21
** 55 83 55 83 AV5	NI	NDD	12
** 54 85 54 85 AV5	NI	28	24
52 86 AV3	NI	NF NF 16 NF	15
* 54 88 54 88 AV5		28	
* 43 90 43 90 AV3 43 90 AV4	NI NI	22 23	26 28
* 43 91 43 91 AV2 43 91 AV3 43 91 AV5	NI N NI N	DD	21 14 11

* UNIT 1 STEAM GENERATOR NO. 2 *

ROW CLMN LOCATIO. IR1 1R2 1R3 1R4 1R5 1R6 1R7 1R8 1R9 1R10 GRTHRTE 新紀期 密維維維 医脊髓性神经球炎 熱毒素 物斑斑 数条款 数化学 新香菜 新松菜 新新斯 新新斯 新新州 的复数多数数据数

10 24
10
24
10
UDS
12
19
12
33
22
12

GROWTH RATE * UNIT 1 STEAM GENERATOR NO. 3 *

ROW CLMN LOCATION 1R1 1R2 1R3 1R4 1R5 1R6 1R7 1R8 1R9 1R10 GRTHRTE 独烈的 植物形式 植食品医胃炎性炎 食物物 新拉斯 數定數 新拉利 医乳腺 數數數 微点的 医性乳 医乳腺 医乳腺 医乳腺 医乳腺性脓肿管

**	17	10			
17	10	7H+1.30	NI	37	MBM
**	38				
38	16	AV2	NI	15	19
38	16	AV5	NI	14	17
38	16	AV6	NI	24	26
**	39	17			
39	17	AV3	NI	16	19
39	17	AV4	NI	NF	12
39	17	AV6	NI	16 NF NF	11
**	40	17 AV3 AV4			
40	17	AV3	NI	33	39
40	17	AV4	NI	12 NF	17
40	17	AV5	NI	NF	15
40	17	AV6	NI	NF	12
**	37	18			
37	18	1E 7H+1.07	NI	UDS	UDS
**	42	27			
42	27	AV5	NI	NDD	15
**	50	28 AV4 AV5 AV6			
50	28	AV4	NI	24	27
50	28	AV5	NI	24 13	19
50	28	AV6	NI	NF	13
**	40	30			
40	30	AV5	NI	20	18
**	39	35			
39	35	35 AV3	NI	13	INR
**	40	35			
40	35	AV4	NI	NF	12
**	45	40			
45	40	AV5	NT	20	14
			114	20	**
**	30	42			
30	42	AV6+16.94	NI	38	INR
**	42	43			
42	43	AV5	NI	19	12

* GROWTH RATE *

* UNIT 1 STEAM GENERATOR NO. 3 *

**	35 44	44 AV6+17.57	NI	35	UDS
** 42 42 42	42 50 50 50	AV2 AV3	NI NI NI	NF 13 22	16 INF 24
**	34 51	51 TSH+5.23	NI	11	NI
** 44	44 51	51 7H+4.97	NI	31	34
** 48	48 54	54 AV6	NI	18	MBM
** 45 45 45 45	45 58 58 58 58	58 AV2 AV3 AV4 AV6	NI NI NI	NF 32 25 NF	18 31 22 17
45	45 59		NI	19	?2
**	21				
45	45 62 62		NI NI	10	15 23
55	55 66 66	7C+7.69 7H+6.74	NI NI	UDS UDS	NI NI
** 59	59 66		NI	UDS	NI
45	45 67		NI	28	MDM

* GROWTH RATE *

* UNIT 1 STEAM GENERATOR NO. 3 *

** 55	55 67	67 7C+7.33	NI	UDS	NI
** 58	58 67	67 7H+6.65	NI	UDS	NI
** 55	55 68	68 7C+7.29	NI	UDS	NI
		72 7H+3.38	NI	UDS	NI
49	72	72 AV3		25	
		73 7H+6.63			
97.67	55 82 82 82 82	82 AV2 AV3 AV4 AV5	14.7	11 17 NF 16	15
55 55	55 83 83 83	AV5	NI	26 19 16	27
53		84 AV3 AV4	NI NI	13 17	15 21
**	21 85	85 7C-1.15	NI	UDS	NDD
** 50	50 85	85 6H+25.15	NI	18	NI
	54 86		NI	12	18
** 53 53	53 88 88	88 AV4 AV5	NI NI	23 NF	30 16

* GROWTH RATE *
* UNIT 1 STEAM GENERATOR NO. 3 *

**	EA	88			
54	88	AV1	NI	11	5 - 7
**	45	92			
45	92	AV5	NI	12	INR
**	49	94			
49	94	AV4 AV5	NI	17	25 21
			14.1	17	6.1
**	50	94 AV3 AV5	417	515	10
50	94	AV3	NI	NF	15
50	94	AV6	NI	NF	14

**	47	97 AV3			
47	97	AV3 AV4	NI	NF	14
71.7	97	WAN	NI	21	21
**	48	97			
48	48 97	AV3	NI	23	30
48	97	AV4	NI	19	24
**	47	98			
47	98	98 AV3	NI	17	24
47	98	AV4	NI	NF	19
47	98	AV5	NI	14	25
**	47	99			
47	99	AV4	NI	16	23
47	99	AV5	NI	15	25
**	31	101			
31	101	70-0.67	NI	UDS	NI
**	41	102			
		AV2	NI	NF	12
		AV3			24
41	102	AV4	NI	11	16
**	44	102			
		AVA	NI	12	INR
**	39	104			
		AV5	NI	11	14

* GROWTH RATE * UNIT 1 STEAM GENERATOR NO. 3 *

41	41 105 105 AV1 105 AV2		16 19	
	39 107 107 AV2	NI	15	15
	36 110 110 AV5	NI	13	14
32 32	32 112 112 AV1 112 AV5 112 AV6	NI	NDD NDD NDD	20

15 to

* GROWTH RATE *

* UNIT : STEAM GENERATOR NO. 4 *

ROW CLMN LOCATION 1R. 1R2 1R3 1R4 1R5 1R6 1R7 1R8 1R9 1R10 GRTHRTE

** 27 09 27 09 AV5 NDD 14 14 ** 24 18 24 18 3H+1.10 10 MBM NI ** 17 23 17 23 7C+1.58 NI UDS NI ** 17 24 17 24 7C+1.70 NI UDS NI ** 05 31 05 31 3H+11.78 17 MBM NI ** 31 35 31 35 5C+28.10 16 INR NI ** 37 35 37 35 AV4+8.30 16 MBM NI ** 41 36 41 36 3H+15.40 21 MBM NDD ** 44 39 44 39 5H+1 .74 26 MBM NDD ** 50 40 50 40 AV2 NI NF 12 50 40 AV3 NI 14 19 50 40 AV4 NI 21 27 50 40 AV5 NI NF 11 ** 43 42 43 42 5C+0.70 19 INF NI ** 20 43 20 43 5H+16.65 NI UDS NI ** 45 60 45 60 4C+22.01 13 14 NI ** 35 61 35 61 1H+14.40 17 MBM NI ** 47 64 47 64 7C+5.37 NI UDS NI

GROWTH RATE * UNIT 1 STEAM GENERATOR NO. 4 *

**	50	72			
50	12	5C+15.20			
**	57	75			
57	75	75 3H+14.05	NDD	26	MBM
**	52	77			
52	77	77 5H+2.60	18	MBM	NI
**	11	78 TSC+39.60 TSH+32.40			
11	78	TSC+39.60	15	INR	NI
			19	INR	NI
**	40	87		2.0	4
40	87	AV4	NDD	10	15
40	6/	AV4 AV5	NDD	11	16
**	44	96 AV3 AV4 AV6			
44	96	AV3	NI	21	17
44	96	AVA	NI	11	15
			NI	10	10
**	37 1	101			
37	101	AV5+4.20	17	MBM	NI
**	42 1	02			
42	102	AV3 AV4	18	17	19
			28	17	18
**	43]	02			
43	102	AV3 AV4	19	22	23
			NF	17	24
**	44 1	02			
44	102	AV3 AV4	25	35	39
44	102	AV4 AV5		18	
	102	WAD	10	28	29
**	38 1				
38	107	AV3	NF		17
	107	AV4 AV5		12	13
			10	18	21
**	37 1				
37	108	AV3	NI	19	
3/	108	AV5	NI	NF	15

GROWTH RATE

* UNIT 1 STEAM GENERATOR NO. 4 *

ROW CLMN LOCATION 1R1 1R2 1R3 1R4 1R5 1R6 1R7 1R8 1R9 1R10 GRTHRTE

** 27 113 27 113 6C+36.35 NI 24 NF

** 21 117 21 117 2C+25.90 17 INR NI

** 17 119 17 119 5H+26.10 16 INR NI 4 41 4

GROWTH RATE * UNIT 2 STEAM GENERATOR NO. 1 *

**	30	10 AV6	10	13
**	31 12		12	INR
** 37	37 15	15 BPH+10.60	18	NI
36	36 17 17	AV2 AV6	NF 13	14 14
** 42	42 20	20 AV2	NDD	12
**	34 25	25 AV1	15	15
** 25	25 26	26 1C+0.00	DI	мвм
** 48 48	48 26 26	26 AV3 AV4	NF 17	12 18
**	48 28	28 AV2	NDD	10
** 50 50 50	50 28 28 28	28 AV3 AV4 AV6	13 NF NF	17
**	53	33 AV4	13	14
** 54 54	54 41 41	41 6C+26.25 6C+27.24	27 31	23 29
57	57 ! 53	53 AV6	NDD	11
47	47 65	55 7C-1.12	28	29

* GROWTH RATE * UNIT 2 STEAM GENERATOR NO. 1

ROW CLMN LOCATION 2R1 2R2 2R3 2R4 2R5 2R6 2R7 2R8 2R9 2R10 GRTHRTE

** 54 86 54 86 1H+1.53 17 17

** 42 104 42 104 AV2 NDD 14

** 27 105 27 105 TSC+10.45 25 MBM * GROWTH RATE * UNIT 2 STEAM GENERATOR NO. 2 *

** 34 12 34 12 7H+3.51	RPC	UDS
** 39 17 39 17 7H+2.49	16	NDD
** 42 19 42 19 AV3	13	INR
** 43 20 43 20 AV2	NDD	
** 43 32 43 32 7C+0.44	12	NI
** 54 38 54 38 AV4		
** 39 46 39 46 6H+36.67		
** 30 47 30 47 6H+23.22	14	NI
** 59 55 59 55 AV1 59 55 AV4	15 14	11 11
** 43 64 43 64 AV1	20	17
** 47 65 47 65 AV4	16	12
** 07 66 07 66 6H+17.84	16	NI
** 59 66 59 66 AV1	15	INF
** 50 67 50 67 AV2 50 67 AV4	16 NF	24 16
** 58 69 58 69 AV4	МВМ	14

* GROWTH RATE * UNIT 2 STEAM GENERATOR NO. 2 *

		71 6H+22.05	18	INR
	53 86 86 86	86 AV3 AV4 AV5	NF	12 12 29
	41 91	91 6H+10.11	13	NI
** 25	25 97	97 6H-4.02	10	NI
48		98 AV3 AV4	NF 33	

* GROWTH RATE

* UNIT 2 STEAM GENERATOR NO. 3 *

34	34 13	13 AV5	13	11
**	32 15	15 AV2	NDD	15
** 32	32 16	16 AV2	14	INR
** 34	34 17	17 6C+35.03	29	28
** 44 44	44 21 21	AV2 AV4	14 17	12 14
** 32 32	32 22 22	22 6C+18.14 6C+19.28	20	19
	45 22			14
47 47	47 24 24	24 AV2 AV4	13 11	15 11
** 47	47 25	25 AV2	14	16
** 49	49 39	39 6H+33.97	32	30
31	31	61 2C+23.19	10	NI
38	38 62	62 AV6+4.73	20	ANR
49	49 65	65 6H+10.67	12	NI
**	47	66 6H+11.51		
41	41 72	72 6H+14.77	10	NI

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4 44 4

* GROWTH RATE *

* UNIT 2 STEAM GENERATOR NO. 3 *

ROW CLMN LOCATION 2R1 2R2 2R3 2R4 2R5 2R6 2R7 2R8 2R9 2R10 GRTHRTE

** 41 80 41 80 6H+38.70 14 NI * GROWTH RATE * UNIT 2 STEAM GENERATOR NO. 4 *

ROW CLMN LOCATION 2R1 2R2 2R3 2R4 2R5 2R6 2R7 2R8 2R9 2R10 GRTHRTE WHE WHEN MUNEUPOL WHE RED NOW MAK MUS MAN DER DER MUN SAME REDUCEDA

**	07	05 5H+30.33	07	NI
**	13	15 5H+20.15		
44	21	21 AV2 AV3 AV5 AV6	17 NF	12 18 12 11
**	01 23	23 TSC+5.63	23	MBM
48	25	25 AV3 AV4 AV5	NDD NDD NDD	16 12 19
** 52	52 33	33 AV5	20	20
** 54 54	54 35 35	35 AV5 AV6	NDD NDD	21 13
**	54 37	37 AV6	NDD	12
**	35 38	38 6H+37.78	29	32
** 53 53	53 43 43	43 AV3 AV5	18 NF	21 13
** 52	52 50	50 TSH+11.96	23	МВИ
		55	NF	
44	44 59	59 AV3	NDD	11

* GROWTH RATE *

* UNIT 2 STEAM GENERATOR NO. 4 *

ROW CLMN LOCATION 2R1 2R2 2R3 2R4 2R5 2R6 2R7 2R8 2R9 2R10 GRTHRTE

**	39 64	64 AV3	16	20
**	53 66	66 5C+1.36	16	NI
** 50	50 67	67 AV4	NDD	18
** 59	59 67	67 AV3	17	16
51 51 51 51	73 73	73 AV2 AV3 AV4 AV5 AV6	14	22
**	31 83	83 4H+32.55	23	MBM
43	43 83 83	83 AV3 AV4	NF 16	
		83 AV6+1.47	15	NI
			NDD	
		91 7H+4.57	RPC	UDS
30	94	94 6H+37.00	10	NI
22		7H+1.98		
**	21 115	115 7C+0.71	UDS	UDS
**	10 121	121 5C+34.67	UDS	UDS

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PLUGGED TUBES

	J SG	# RO	W CLM	OTG	DATE			% ***	HEAT NUMBER	REMARKS
**	k UN	28 28 28	SG 1 37 37 37	1 1 3	10-88 10-88 09-91	4 C 5 H	38.00 07.00	39	NX2387 NX2387 NX6977HK	I-600
4.4	UN	IT 1	SG 2						NX6441HK NX6441HK NX6441HK NX6441HK	I-690 I-690 I-690 I-690
**		T 1	SG 3 33 108	0					UNKNOWN	
** 1 1 1	4	T 1 2 32 32 32	SG 4 23 67 77 82	0					UNKNOWN UNKNOWN UNKNOWN UNKNOWN	AT W SHOP AT W SHOP AT W SHOP AT W SHOP
** 2 2 2 2 2	UNI 1 1 1	1 27	SG 1 1 1 42 42	P	08-88	TEC	05.41	51 SM	NX2387 NX2387 NX2387 NX2387	1-600
2 2	2	51 2 5	SG 2 32 48 48 103	0					UNKNOWN UNKNOWN UNKNOWN UNKNOWN	AT W SHOP AT W SHOP AT W SHOP
**		T 2	SG 3 102	0					UNKNOWN	AT W SHOP
** 2 2 2 2 2 2 2 2 2 2 2 2 2	UNI 4 4 4 4 4 4 4	T 2 3 12 8 16 48 23 22 24	SG 4 35 36 56 57 64 67 90 93	0 0 0 0 0 0 0					UNKNOWN UNKNOWN UNKNOWN UNKNOWN UNKNOWN UNKNOWN UNKNOWN UNKNOWN	AT W SHOP