NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Turkey Point Unit 4	0 5 0 0 0 2 5 1	1 OF 0 1

TITLE (4)
Technical Specification - Containment Integrity

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
									N/A	0 5 0 0 0
0 4	1 1	8 5	8 5	— 0 0 8	— 0 0	0 5	1 3	8 5	N/A	0 5 0 0 0

OPERATING MODE (9)	N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	1 0 0	20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	X 50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
	AREA CODE
R. D. Hart, Licensing Engineer	3 0 5 — 2 4 5 - 2 9 1 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPRDS	
X	J M	P S V	M 1 2 0	Y							
X	J M	P S F	C 6 8 5	Y							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

Event:

On April 11, 1985, while Unit 4 was at 100% power, a small leak was discovered at a fitting on a test connection between containment and containment isolation valve, CV-4-956A, located on the pressurizer steam space sampling line. Containment boundary valve, CV-4-951, located just inside containment on that line was cycled in an attempt to stop the leak. CV-4-951 would not close when given a close signal from the control room. This sequence of events was interpreted as a loss of containment integrity as defined in Technical Specification 1.5. During this event, CV-4-956A remained operable and closed which would prevent a flow path to outside containment.

Cause of Event:

The cause of the event was the test connection leakage and the malfunction of CV-4-951. The reason for the malfunction of CV-4-951 will be investigated when the valve is repaired during the next unit outage of sufficient duration.

Corrective Actions:

Corrective Actions include the following:

1) The power supply to CV-4-951 was disconnected and the valve was verified to have failed closed as designed. Manual isolation valve 4-977A upstream of CV-4-951 was closed also.
2) The test connection was repaired and the leakage stopped.
3) Due to ALARA considerations, CV-4-951 and 4-977A will remain closed, as the system is not required at power operation. CV-4-951 will be inspected and repaired during the next outage of sufficient duration.

The Plant Nuclear Safety Committee reviewed a written evaluation of this event and configuration of CV-4-951 and 4-977A and concluded that operation in this manner is acceptable and would not affect the health and safety of the public. No radiological releases were detected during this event. Significant event notification was made to the NRCOC via ENS pursuant to 10 CFR 50.72(b)(1)(ii). Similar occurrences: LER 250-85-002, 251-85-001, 250-84-034, 250-84-031, 250-84-020, 251-84-009, and 250-83-016

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FLORIDA POWER & LIGHT COMPANY

MAY 1 3 1985

L-85-192

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Gentlemen:

Re: Reportable Event 85-008
Turkey Point Unit 4
Date of Event: April 11, 1985
Technical Specification -
Containment Integrity

The attached Licensee Event Report is being submitted pursuant to the requirements of 10 CFR to provide notification of the subject event.

Very truly yours,

J. W. Williams, Jr.
Vice President
Nuclear Energy Department

JWW/JA/awt/T14:5

Attachment

cc: Dr. J. Nelson Grace
Harold F. Reis, Esquire

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