

LICENSEE EVENT REPORT (LER)

| | | |
|---|--|----------------------------|
| FACILITY NAME (1) Waterford 3 Steam Electric Station | DOCKET NUMBER (2) 0 5 0 0 0 3 8 2 | PAGE (3) 1 OF 0 3 |
|---|--|----------------------------|

TITLE (4)
Automatic Actuation of Reactor Protective System

| EVENT DATE (5) | | | LER NUMBER (6) | | | REPORT DATE (7) | | | OTHER FACILITIES INVOLVED (8) | | |
|----------------|-----|------|----------------|-------------------|-----------------|-----------------|-----|------|-------------------------------|--|-------------------|
| MONTH | DAY | YEAR | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | MONTH | DAY | YEAR | FACILITY NAMES | | DOCKET NUMBER(S) |
| 0 | 5 | 2 | 4 | 8 | 5 | 8 | 5 | 0 | N/A | | 0 5 0 0 0 |
| | | | | | | | | | N/A | | 0 5 0 0 0 |

OPERATING MODE (9) 1

POWER LEVEL (10) 0 | 1 | 1

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

| | | | |
|--|---|---|--|
| <input type="checkbox"/> 20.402(b) | <input type="checkbox"/> 20.405(c) | <input checked="" type="checkbox"/> 50.73(a)(2)(iv) | <input type="checkbox"/> 73.71(b) |
| <input type="checkbox"/> 20.405(a)(1)(i) | <input type="checkbox"/> 50.36(e)(1) | <input type="checkbox"/> 50.73(a)(2)(v) | <input type="checkbox"/> 73.71(c) |
| <input type="checkbox"/> 20.405(a)(1)(ii) | <input type="checkbox"/> 50.36(e)(2) | <input type="checkbox"/> 50.73(a)(2)(vii) | OTHER (Specify in Abstract below and in Text, NRC Form 366A) |
| <input type="checkbox"/> 20.405(a)(1)(iii) | <input type="checkbox"/> 50.73(a)(2)(ii) | <input type="checkbox"/> 50.73(a)(2)(viii)(A) | |
| <input type="checkbox"/> 20.405(a)(1)(iv) | <input type="checkbox"/> 50.73(a)(2)(iii) | <input type="checkbox"/> 50.73(a)(2)(viii)(B) | |
| <input type="checkbox"/> 20.405(a)(1)(v) | <input type="checkbox"/> 50.73(a)(2)(iii) | <input type="checkbox"/> 50.73(a)(2)(x) | |

LICENSEE CONTACT FOR THIS LER (12)

| | |
|--|---|
| NAME T. Smith, Maintenance Superintendent | TELEPHONE NUMBER 5 0 4 4 6 4 - 3 1 3 8 |
|--|---|

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

| CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NPRDS | CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NPRDS |
|-------|--------|-----------|--------------|---------------------|-------|--------|-----------|--------------|---------------------|
| | | | | | | | | | |
| | | | | | | | | | |

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15) MONTH DAY YEAR
 0 | 8 | 3 | 0 | 8 | 5

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

ABSTRACT

At approximately 1433 hours on May 26 1985, Waterford 3 Steam Electric Station was at 11% reactor power when Operations Personnel were in the process of warming up the Main Turbine when Turbine Bypass Valve MS-319A suddenly opened (instead of modulating open). This in turn precipitated a swell in Steam Generator water levels causing a reactor trip. Plant conditions were subsequently stabilized in mode 3 (hot standby).

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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| FACILITY NAME (1) Waterford 3 Steam Electric Station | DOCKET NUMBER (2) 0 5 0 0 0 3 8 2 8 5 - 0 2 2 - 0 0 | LER NUMBER (6) | | | PAGE (3) | | |
| | | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | | | |
| | | | | | 0 2 | OF | 0 3 |

TEXT (If more space is required, use additional NRC Form 366A's) (17)

NARRATIVE

On May 24, 1985, Waterford 3 Steam Electric Station was at 11% reactor power when Operations Personnel were warming the Main Turbine as described in procedure OP-5-007, Main Turbine and Generator. The Steam Bypass Control System was in automatic with the secondary steam pressure (950 psia) less than the setpoint on the master controller. As steam pressure was increased above the setpoint, Operations Personnel expected the Turbine Bypass Valve, MS-319A, to modulate open to compensate for the increase in steam pressure. However, MS-319A failed to open and the output signal for the second Turbine Bypass Valve, MS-320A, started to increase in an effort to modulate the valve open. Suddenly, the controller output for MS-319A increased to 60% demand and the valve popped open. The sudden opening of MS-319A caused a swell in Steam Generator water levels, and at approximately 1433 hours on May 24, 1985, Waterford 3 experienced an uncomplicated reactor trip on high Steam Generator water level. Operations Personnel immediately entered Emergency Procedures OP-902-000, Emergency Entry Procedure and OP-902-001, Uncomplicated Reactor Trip Recovery Procedure. Plant conditions were subsequently stabilized in mode 3 (hot standby).

SAFETY CONSEQUENCES AND IMPLICATIONS

The above event resulted in an actuation of the Reactor Protective System during initial Startup testing in which no primary system parameters were exceeded. Since the Control Element Assemblies and the Reactor Protective System functioned as designed, the event in no way placed Waterford 3 in a degraded safety condition.

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

CORRECTIVE ACTION

During the performance of procedure SIT-TP-707, Atmospheric Steam Dump and Steam Bypass Control Valve Capacity Check, several of the Turbine Bypass Valves experienced problems similar to that described above. Several attempts at troubleshooting failed to identify any problems (for example, the internals of valve MS-320C were completely removed and no evidence of mechanical binding was evident). Each of the subject valves were satisfactorily stroked on May 23, 1985. However, the valves were stroked at cold conditions. The valves in question will be tested again when Waterford 3 returns to approximately 50% reactor power (Waterford 3 has been in an outage since May 31, 1985) in an effort to determine if the valves require additional attention.

SIMILAR EVENTS

None

PLANT CONTACT

T. Smith, Maintenance Superintendent, 504/464-3138



LOUISIANA
POWER & LIGHT

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NEW ORLEANS, LOUISIANA 70174-8008 • (504) 386-2345

June 21, 1985

W3P85-1416
A4.05

Director, Office of Nuclear Reactor Regulation
ATTENTION: Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: Waterford 3 SES
Docket No. 50-382
License No. NPF-38
Reporting of Licensee Event Report

Dear Sirs:

Attached is Licensee Event Report Number LER-85-022-00 for the Waterford 3 Steam Electric Station. This Licensee Event Report is submitted per 10CFR50.73(a)(2)(iv).

Very truly yours,

K.W. Cook
Nuclear Support & Licensing Manager

KWC:GEW:sms

Attachment

cc: R.D. Martin, G.W. Knighton, D.M. Crutchfield, NRC Resident Inspectors
Office, INPO Records Center (J.T. Wheelock), B.W. Churchill,
W.M. Stevenson

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