

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) DIABLO CANYON, UNIT 1	DOCKET NUMBER (2) 0 5 0 0 0 2 7 5	PAGE (3) 1 OF 0 2
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TITLE (4)
REACTOR TRIP AND SAFETY INJECTION

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0 5	2 0	8 5	8 5	0 1	0 0	0 6	1 8	8 5			0 5 0 0 0
											0 5 0 0 0

OPERATING MODE (9) 1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)									
POWER LEVEL (10) 0, 5, 5	20.402(b)	<input checked="" type="checkbox"/>	20.406(c)	<input type="checkbox"/>	50.73(a)(2)(iv)	<input checked="" type="checkbox"/>	73.71(b)	<input type="checkbox"/>		
	20.406(a)(1)(i)	<input type="checkbox"/>	50.38(c)(1)	<input type="checkbox"/>	50.73(a)(2)(v)	<input type="checkbox"/>	73.71(c)	<input type="checkbox"/>		
	20.406(a)(1)(ii)	<input type="checkbox"/>	50.38(c)(2)	<input type="checkbox"/>	50.73(a)(2)(vii)	<input type="checkbox"/>	<input checked="" type="checkbox"/> OTHER (Specify in Abstract below and in Text, NRC Form 366A)			
	20.406(a)(1)(iii)	<input type="checkbox"/>	50.73(a)(2)(i)	<input type="checkbox"/>	50.73(a)(2)(viii)(A)	<input type="checkbox"/>				
	20.406(a)(1)(iv)	<input type="checkbox"/>	50.73(a)(2)(ii)	<input type="checkbox"/>	50.73(a)(2)(vii)(B)	<input type="checkbox"/>				
20.406(a)(1)(v)	<input type="checkbox"/>	50.73(a)(2)(iii)	<input type="checkbox"/>	50.73(a)(2)(ix)	<input type="checkbox"/>					

Special Report

LICENSEE CONTACT FOR THIS LER (12)

NAME DAVID P. SISK, REGULATORY COMPLIANCE ENGINEER	TELEPHONE NUMBER
	AREA CODE: 8 0 5 NUMBER: 5 9 5 - 7 3 5 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15)	MONTH DAY YEAR
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ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At 0520 PDT, May 20, 1985, with Unit 1 in Mode 1 (power operation), a reactor trip followed by a safety injection occurred. All automatic equipment responded as designed. The plant was stabilized in Mode 3 (hot standby) in accordance with procedures. All systems and equipment affected by this event were returned to normal operation.

This event was caused by a loose connection to the output circuit breaker for instrument inverter IY-1-2. This resulted in the breaker tripping open producing a reactor coolant pump breaker position trip signal. Since the unit was above P-8 (Loss of Flow Permissive), only one breaker open signal was required to initiate a reactor trip signal. The connections were reterminated and the inverter was returned to normal operation.

To prevent recurrence, the output circuit breaker connections will be rechecked in 30 days and again quarterly until the connections are replaced with an improved method of making terminations.

This was the seventh emergency core cooling system (ECCS) actuation cycle to date that has resulted in the discharge of water into the reactor coolant system.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) DIABLO CANYON UNIT 1	DOCKET NUMBER (2) 0 5 0 0 0 2 7 5	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 5	- 0 1 5	- 0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

At 0520 PDT, May 20, 1985, with Unit 1 in Mode 1 (power operation), a reactor trip followed by a safety injection occurred. All automatic equipment responded as designed. The plant was stabilized in Mode 3 (hot standby) in accordance with procedures. All systems and equipment affected by this event were returned to normal operation.

A loose connection to the output circuit breaker for instrument inverter IY-1-2 (EE)(BU) caused the breaker to trip open. This resulted in the loss of power to the Reactor Coolant Pump (RCP) breaker position indicator (JC)(ZI), which produced an RCP breaker (AB)(BKR) open signal. Since the unit was above P-8 (Loss of Flow Permissive), only 1 RCP breaker open signal was required to produce the reactor trip signal. The safety injection signal resulted from a 1o-1o Tavg signal for all four loops, due to cooldown after the reactor trip, coincident with the high steam flow signals (produced from all four steam lines when bistables in Reactor Protection Set 2 were deenergized on loss of inverter IY-1-2).

The connections were reterminated and the inverter was returned to normal operation. To prevent recurrence, the output circuit breaker connections will be rechecked in 30 days and again quarterly until the connections are replaced with an improved method of making terminations.

This was the seventh emergency core cooling system (ECCS) actuation cycle to date that has resulted in the discharge of water into the reactor coolant system.

PACIFIC GAS AND ELECTRIC COMPANY

PG&E

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JAMES D. SHIFFER
VICE PRESIDENT
NUCLEAR POWER GENERATION

June 18, 1985

PGandE Letter No.: DCL-85-219

Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Re: Docket No. 50-275, OL-DPR-80
Diablo Canyon Unit 1
Licensee Event Report 85-015-00
Reactor Trip and Safety Injection

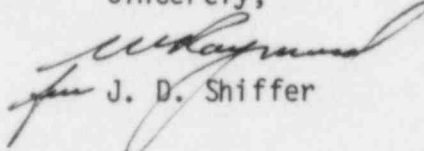
Gentlemen:

Pursuant to 10 CFR 50.73(a)(2)(iv) and as required by Diablo Canyon Unit 1 Technical Specification 6.9.2 and Action Statement b of Technical Specification 3.5.2, PGandE is submitting the enclosed Licensee Event Report/Special Report concerning the inadvertent actuations of Engineered Safety Features (ESF), a reactor trip followed by a safety injection.

This event has in no way affected the public's health and safety.

Kindly acknowledge receipt of this material on the enclosed copy of this letter and return it in the enclosed addressed envelope.

Sincerely,


J. D. Shiffer

Enclosure

cc: J. B. Martin
Service List

IE22
1/1