REFERENCES AND A CONTRACTORS			
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January 21, 1993

William J. Cahill, Jr. Group Vice President

U. S. Nuclear Regulatory Commission Attn: Document Control Desk Washington, DC 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES) - UNIT 2 DOCKET NO. 50-446 ADVANCE FSAR SUBMITTAL SEISMIC CATEGORY II PIPING AND SUPPORTS LOCATED IN A NON-CATEGORY I BUILDING

- REF: 1) NUREG-0797, "Safety Evaluation Report," Supplement 22 (SSER 22)
 - 2) TU Electric letter logged TXX-92063 from William J. Cahill, Jr., to the NRC dated March 4, 1992

Gentlemen:

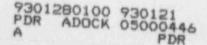
Attached is an advance FSAR change to reclassify a portion of the eight (8) inch Heater Drain piping and one pipe support in the Turbine Building from non-seismic to seismic Category II. This change permits one seismic pipe support to be located thirteen inches inside the Turbine Building. This seismic pipe support is a seismic/non-seismic anchor for a seismically qualified non-ASME, high energy Heater Drains piping system.

The reclassified portion of the Heater Drains piping and subject pipe support are completely supported by a Seismic Category 1 wall which separates Room 113 from the non Category I Turbine Building. An analysis and an engineering evaluation have been performed which demonstrated that the Turbine Building will not have unacceptable interactions with the reclassified piping during and after a seismic event.

The attachment is organized as follows:

- A marked-up copy of the revised FSAR pages (additional pages immediately preceding and/or following the revised pages are provided if needed to understand the change).
- 2. A description/justification of each FSAR change.
- 3. A copy of related SER/SSER sections.

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400 N. Olive Street L.B. 81 Dallas, Texas 75201

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This change will be included in a future FSAR Amendment. If you have any questions regarding this submittal, please contact Mr. Carl B. Corbin at (214) 812-8859.

Sincerely,

William J. Cahill, Jr.

Marshall By

J. S. Mafshall Generic Licensing Manager

CBC/tg Attachment

c - Mr. J. L. Milhoan, Region IV Resident Inspectors, CPSES (2) Attachment to TXX-93037 Page 1 of 14

Attachment to TXX-93037

1.	Marked-up copy of FSAR pages	pages 2 th
2.	Description/justification	pages 8 ar
3	Pelated STP sections	

pages 2 through 7 pages 8 and 9 pages 10 through 14 Attachment to TXX-93037 CPSES/FSAR Page '2 of 14 35. Miscellaneous Handling Equipment 52 DRAFT 40. Plant Gas System 7 42. Tornado Venting Components 7 Potable and Sanitary Water System 45. 12 49. Pipe Whip Restraints 12 51. Uninterruptible Power Supply (UPS) Area Air-Conditioning System 66 3.2.1.1.3 Seismic Category I Electrical Systems and Components 7 All, or portions, of the following electrical systems or components 7 are seismic Category I as described in Appendix 17A and Table 17A-1: 17A SYSTEM 7 NO. 7 SYSTEM 37. Electrical Equipment 7 38. Radiation Monitoring System 7 41. Instrumentation and Control Equipment 7 3.2.1.1.4 Structures and Systems of Mixed Category None of the plant structures are classified as partially seismic 59 Category I; however, certain structural items within seismic Category I structures are classified as seismic Category II or non-seismic as appropriate. See Table 17A-1, item 36, for typical structural classifications. The boundaries of seismic Category I portions of systems are shown on the piping and instrumentation diagrams in appropriate sections of the FSAR. A seismic category II pipe located 85 inside a non-seismic building to described in Section 3.2.2.d. are piping segments 3.2-5 Amendment 85 May 29, 1992

Attachment to TXX-93037 Segments, which are listed in FSAR Section 3.78.2.8 as Page 3 of 14 portion of the Steam Generator Blowdown Kigh energy line-85 piping located in the Turbine Building, is designated class 5 piping and classified as seismic Category II. This These segments are piping to seismically analyzed for break postulation, and seismic Additional analyses are performed to demonstrate that Qualification interactions of these piping, with non-Category I unarceptable structures/components will not occur during a seismic event.

Class 5 lines, which are determined as not reducing the functioning of the systems and components described above to an unacceptable degree, and Class G lines are fabricated and installed in accordance with applicable industry codes and standards.

All, or portions of the systems or components that are Seismic Category II are described in Appendix 17A and Table 17A-1.

Radioactive Waste Management System (RWMS)

The RWMS designation is used to identify the boundaries of the radioactive waste management system on applicable CPSES flow diagrams, as the RWMS does not match the functional

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66

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66

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line

Dortion of the Heater Drain

5

-069-302

N

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- CH-8

portion of the Steam Generator Blowdown

(8-58-2-060-1302-5

3.78.2.8 Interaction of Non-Category I Structures

with Seismic Category I Structures

A number of structures such as the Turbine Building, the Switchgear Buildings, the Circulating Water Intake and Discharge Structures, the Maintenance Building, and the Administration Building are designated as non-Category I.

CPSES/FSAR

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The only non-Category I structures which are adjacent to any seismic Category I structure are the Turbine Building and the Switchgear Buildings. These structures do not share a common mat with the adjacent seismic Category I structure, and all structures are founded on firm rock. Therefore, there is no possible interaction of non-Category I structures with seismic Category I structures resulting from seismic motion. Sufficient space is provided between the Turbine and Switchgear Buildings and the adjacent seismic Category I structure so as to prevent contact because of deformations occurring in the structures during a seismic event.

The possibility of structural failure during a seismic event is considered for the Turbine Building. Structural failure in the direction of the adjacent seismic Category I structure is prevented by the bearing of the mezzanine and operating floor slabs on the concrete turbine generator pedestal. The Switchgear Buildings are design to withstand a seismic event equal to the SSE.

The seismic Category II portion of the Steam Generator Blowdown high energy line (8-SB-2-060-1302-5) located inside the Turbine Building, and attached to a seismic Category I structure is shown by analysis to remain undamaged by non-Category I structures and components during a seismic event. The piping Segments involved are : 9

54

78

85

Non-Category I equipment and components located in seismic Category I buildings are investigated by analysis or testing, or both, to ensure that under the prescribed earthquake loading, structural integrity is maintained, or the non-Category I equipment and components do not adversely affect the integrity or operability, or both, of any

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TABLE 17A-1

(Sheet 14)

LIST OF QUALITY ASSURED STRUCTURES, SYSTEMS AND COMPONENTS

		Applicable					
	Safety	Code or	Coda	Seimaic	Quality	Reference	
System and Components	Class (7)	Standard (12)	Class	Category	Assurance	Section	Remarka
Theck valves for accumulator tanks	3	ASME III	3	I	Note 32, A	10.3	Note 79
						Q260.1	
fubing and supports (between check	3	ASME III	3	I	Note 32,A	3.9B	Note 41
valves upstream of air accumulator							
and AOV)							
iteam generator blowdown	2	ASME III	2	1	Note 26,A	10.3	
system piping							
Steam flow restrictor (integral to	2	ASME III	2	I	Note 4,A	10.3, 5.4.4	
steam generator)							
ain steem isolation valves	2	ASME III	1	I	Note 26, A	10.3	Notes 8, 79
						Q260.1	
ain steam isolation bypass valves	2	ASME III	2	I	Note 26,A	10.3	Notes 8, 79
and bypass valve piping						Q260.2	
eflective insulation assemblies	NRS	Mfrs Stds	-	II	Note 27,B	6.2.2	
iping and valves	2	ASME III	2	I	Note 26,A	10.3	Note 79
						Q260.1	
iping and valves	3	ASME III	э	I	Note 26,A	10.3	Note 79
						Q260.1	
rifices	3	ASME III	з	Ι	Note 26,A	10.3	
upports for Class 2 Piping	2	ASHL III	2	I	Note 27,A	3.9B	
upports for Class 3 Piping	3	ASME III	3	I	Nots 27,A	3.98	
lass 5 Piping and supports	MINS	ANSI 831.1	-	II/NONE	Note 44,8	3.78	Note 81
3. Auxiliary Feedwatar System							
Sotor-driven suziliary feedwater pumps	3	ASME III	3	τ	Note 26,A	10.4.9	Note la
furbine-driven auxiliary feedwater pump	3	ASME III	3	r	Note 26, A	10.4.9	Note le
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LIST OF QUALITY ASSURED STRUCTURES, SYSTEMS AND COMPONENTS

		Applicable					
	Safety	Code or	Code	Seimmic	Quality	Reference	
System and Components	Class (7)	Standard (12)	Class	Category	Assurance	Section	Remarks
b. Rafrigarant piping 6 tubes	3	ASTM B42	-	I	Note 26, A	Fig. 9.4-15	Note 21
		ASTN B88			Note 27, 50,A		
s. Water Side	3	ASHE III	3	I	Note 26,&	Fig. 9.4-15	
Campers and supports	3	Mfrs Stds	-	I	Note 26, 32, A	Fig. 9.4-15	Note 21
Juctwork and supports	3	Mfrs Stds	-	I	Note 26, 32,A	Fig. 9.4-15	Note 21
Boostar Return Fans	3	Mfr: Stde	-	I	Note 26,A	Fig. 9.4-15	Note 21
52. Turbine Flant Cooling							
Water System							
lass 5 piping and supports	NHS	ANSI B31.1		II/None	Note 44,8	10.4.12	
 Condensats Polishing System 							
lass 5 piping and supports	NNS	ANSI B31.1		II/None	Note 44,8	10.4.6	
if. Condenser Vacuum and							
Wate boz Priming System							
lass 5 piping and supports	NINS	ANSI 831.1		II/None	Note 44,B	10.4.2	~~~~
5. Hestor Drains System						5	1+ 01)
lass 5 piping and supports	NNS	ANSI 831.1		II/None	Note 44,B	10.4.11 (Vote 81)
6. Chemical Food System						-	\sim
lass 5 piping and supports	HINS	ANSI B31.1		II/Mone	Note 44,B	₽ig. 10.3-1	
7. Security Systems							
arriers/doors	N/A	Mfre Stde	+	NONE	Note C	13.6	
ntrusion Detection/	NIE	Mfrs Stds		NONE	Note C	13.6	
Monitoring							
ighting	NIR	Mfrs Stde		NONE	Note C	13.6	
ccess Control System	NIE	Mfr# Stds	14.00	NONE	Note C	13.6	Ame-idment 87

December 18, 1992

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CPSES/FSAR

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(Sheet 60)

LIST OF QUALLTY ASSURED STRUCTURES, SYSTEMS AND COMPONENTS

80	Cables and heat shrinkable cable insulation sleeves are not seismically qualified. However, the basic standards to which cables and heat	8.
	shrintable cable insulation sleeves are procured and their associated installation requirements assure their functioning following a seismic	
	avant. Segments which are listed in FSAR Section 3.78.2.8 as	
0	are 1	
81.	A portion of the Steam Generator Blowdows high energy line piping located in the Turbine Building is designated class 5 piping and the classified	85
as	Seismic Category II even though the location is in a non-seismic building. Additional analyses are performed to demonstrate that interactions of	
	segments with Non-Category I structures and components will not occur during a seismic event. This piping be seismically analyzed for break	
These	postulation, and seismic qualification. These segments are	
have		

B2. Piping and supports for the Diesel Generator exhaust piping the building, from the flaxible connectors to just upstream of the exhaust relief valve, are designed, analyzed, installed and inspected to the requirements of ASME Section III, including Code Case N-253-2. In some of the p-pe support qualifications, part of the material stress allowables are extracted from AMSI B31.1. In all cases, the requirements of Code Case N-253-2 were met or exceeded. The piping and associated supports are exempt from Code Stamping.

AMENDMENT 85 MAY 29, 1992

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CPSES - FINAL SAFETY ANALYSIS REPORT (FSAR) Attachment to AMENDMENT / REVISION 88 TXX-93037 DETAILED DESCRIPTION Page 1 Page 8 of 14 Prefix Page (as amended) Group Description 3.2-5 1 Revises discussion of Structures and Systems of Mixed Category. REVISION 1 Revises sentence to indicate that there is more than one Seismic Category II piping segment located in a non-seismic building. Change Request Number : SA-93-4.1 Commitment Register Number : Related SER : 3.2 SSER : SER/SSER Impact : No 3.2-12 1 Revises discussion regarding piping segments located in the Turbine Building which are designated class 5 piping and classified as seismic Category II. Revision 12 Changes discussion to indicate that there is more that one piping segment located in the Turbine Building which is designated class 5 piping and classified as seismic Category II. The applicable piping segments are listed in Section 3.7B.2.8. Also clarifies that these piping segments are analyzed for seismic qualification and unacceptable interactions with non-category I structures/compenents. Change Request Number : SA-93-4.2 Commitment Register Number : Related SER : 3.2 SSER : SER/SSER Impact : No 3.78-42 1 Revises discussion of Interaction of Non-Category I Structures with Seismic Category I Sturctures. Revision : Adds the specific exception concerning the Unit 2 Heater Drain line piping in the Turbine Building. The Heater Drain piping has been reclassified as Class 5 and redesignated as Seismic Category II to allow the placement of a seismic/non-seismic boundary anchor of a seismically gualified high energy Heater Drains pipe line thirteen inches inside the Turbine Building. Analyses have been performed that demonstrate that the non-Category I structures and components within the Turbine Building will not unacceptably interact with Heater Drain piping during and after a seismic event. Change Request Number : SA-93-4.3 Commitment Register Number : Related SER : 3.7 SSER : SER/SSER Impact : No Table 17A-1 1 See Sheet No(s) :50 Add Note 81 to List of Quality Assured Structures.

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Attachment to CPSES - FINAL SAFETY ANALYSIS REPORT (FSAR) AMENDMENT / REVISION 88 DETAILED DESCRIPTION

Prefix Page (as amended)

Group Description

SER/SSER Impact

1

Revision 18 The specific exception concerning the Heater Drain piping in the Turbine Building has been added to Table 17A-1 via Note 81. The quality assurance requirements for this piping will be the same as for any Class 5, Seismic Category II piping except that the piping is not located in a Seismic Category I structure. Change Request Number : SA-93-4.4 Commitment Register Number : Related SER : SSER :

Table 17A-1

See Sheet No(s) :60 Revises Note 81 to List of Quality Assured Structures, Systems and Components. Revision 1.2

: No

Revises note 81 to indicate that there is more than one piping segment located in the Turbine Building which is designated class 5 piping and classified as Seismic Category II even though located in a nonseismic building. The applicable piping segments are listed in Section 3.7B.2.8.

Change Request Number : SA-93-4.5 Commitment Register Number : Related SER : SSER : SER/SSER Impact : No

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3 DESIGN CRITERIA FOR STRUCTURES, SYSTEMS, AND COMPONENTS

3.1 Conformance With General Design Criteria and NRC Regulations

In Section 3.0 of the FSAR, the applicant presented an evaluation of the design bases against the GDC. In a letter dated February 20, 1981, the NRC staff asked the applicant to provide a compilation which documents that the Comanche Peak Steam Electric Station Units 1 and 2 will comply with the regulations given in 10 CFR Parts 20, 50, and 100. The applicant has not responded to this request for information and this matter remains an open issue.

The staff review of structures, systems, and components relies extensively on the application of industry codes and standards that have been used as accepted industry practice. These codes and standards, as cited in this report, have been reviewed and found acceptable by the staff, and they have been incorporated into the SRP.

3.2 Classification of Structures, Systems, and Components

3.2.1 Seismic Classification

GDC 2, in part, requires that nuclear power plant structures, systems, and components important to safety be designed to withstand the effects of earthquakes without a loss of capability to perform their safety function. These plant features are those necessary to ensure (1) the integrity of the reactor coolant pressure boundary, (2) the capability to shut down the reactor and maintain it in a safe-shutdown condition, or (3) the capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures comparable to 10 CFR Part 100 guideline exposures. The earthquake for which these plant features are designed is defined in 10 CFR Part 100, Appendix A as the safe-shutdown earthquake (SSE). The SSE is based on an evaluation of the maximum earthquake potential and is that earthquake which produces the maximum vibratory ground motion for which structures, systems, and components important to safety are designed to remain functional. Those plant features that are designed to remain functional if an SSE occurs are designated seismic Category I in Regulatory Guide 1.29. This Regulatory Guide is the principal document used in the staff review for identifying those plant features important to safety which, as a minimum, should be designed to seismic Category I requirements. The staff review of the seismic classification of structures, systems, and components (excluding electrical features) of Comanche Peak Units 1 and 2 was performed in accordance with the guidance in SRP Section 3.2.1.

The structures, systems, and components important to the safety of Commanche Peak that are required to be designed to withstand the effects of an SSE and remain functional have been identified in an acceptable manner in Table 17A-1 of the FSAR. Table 17A-1, in part, identifies major components in fluid systems, mechanical systems, and associated structures designated as seismic Category I. In addition, piping and instrumentation diagrams in the FSAR Attachment to TXX-93037 Page 11 of 14

identify the interconnecting piping and valves and the boundary limits of each system classified as seismic Category I. The staff has reviewed Table 17A-1 and the fluid system piping and instrumentation diagrams and has concluded that the structures, systems, and components important to safety of Comanche Peak Units 1 and 2 have been properly classified as seismic Category I items in conformance with Regulatory Guide 1.29, Revision 2. In the review of Section 3.9 of the FSAR, the staff confirmed that acceptable interfaces exist between seismic Category I and nonseismic portions of piping systems. All other structures, systems, and components that may be required for operation of the facility are not required to be designed to seismic Category I requirements. This exclusion includes those portions of Category I systems such as vent lines, fill lines, drain lines, and test lines on the downstream side of isolation valves and portions of these systems that are not required to perform a safety function.

The staff concludes that the Comanche Peak structures, systems, and components important to safety that are designed to withstand the effects of an SSE and remain functional are properly classified as seismic Category I items in accordance with Regulatory Guide 1.29. This constitutes an acceptable basis for satisfying, in part, the requirements of GDC 2, and is, therefore, acceptable.

3.2.2 System Quality Group Classification

GDC 1 requires that nuclear power plant systems and components important to safety be designed, fabricated, erected, and tested to quality standards commensurate with the importance of the safety function to be performed. These fluid-system, pressure-retaining components are part of the reactor coolant pressure boundary and other fluid systems important to safety, where reliance is placed on these systems: (1) to prevent or mitigate the consequences of accidents and malfunctions originating within the reactor coolant pressure boundary, (2) to permit shutdown of the reactor and maintain it in a safeshutdown condition, and (3) to recain radioactive material. Regulatory Guide 1.26 is the principal document used in the staff review for identifying, on a functional basis, the components of those systems important to safety that are Quality Groups B, C, and D. Section 50.55a of 10 CFR Part 50 identifies those American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code Section III, Class 1 components that are part of the reactor coolant pressure boundary (RCPB). Conformance of these RCPB components with Section 50.55a of 10 CFR Part 50 is discussed in Section 5.2.1.1 of this report. These RCPB components are designated in Regulatory Guide 1.26 as Quality Group A. Certain other RCPB components which meet the exclusion requirements of footnote 2 of the rule are classified Quality Group B in accordance with Regulatory Guide 1.26. The staff review of the quality group classification of pressure-retaining components of fluid systems important to safety for Comanche Peak Units 1 and 2 was performed in accordance with the guidance in SRP Section 3.2.2.

FSAR Table 17A-1, in part, identifies the major components in fluid systems such as pressure vessels, heat exchangers, storage tanks, pumps, piping, and valves, as well as mechanical systems such as cranes, refueling platforms, and other miscellaneous handling equipment. In addition, the piping and instrumentation diagrams in the FSAR identify the classification boundaries of the interconnecting piping and valves. The applicant has utilized the American Nuclear Attachment to TXX-93037 Page 12 of 14

Based on the review described above, the staff concludes that the applicant has not met the requirements of GDC 4 regarding pipe breaks. The staff will provide the resolution to the open items described above in a supplement to this report.

3.7 Seismic Design

3.7.1 Seismic Input

The input seismic design response spectra (operating-basis earthquake (OBE) and safe-shutdown earthquake (SSE)) applied in the design of seismic Category I structures and components were developed from numerous real records, following the procedures recommended by Newmark, Blume, and Kapur* and conform to the requirements of Regulatory Guide 1.60, Revision 1, with the exception of those in the 33-Hz to 50-Hz frequency range. In this range, the vertical response spectrum of Regulatory Guide 1.60, Revision 1, differs from the vertical response spectrum used by the applicant. Because this deviation only affects the modes that have low amplification, the effect of this deviation on the results of the analyses of structures and systems is negligible. Similarly, the method recommended by Newmark and his colleagues for the construction of vertical response spectra leads to a slight deviation from the Regulatory Guide 1.60, Revision 1, recommendations for accelerations corresponding to 3.5 Hz. The magnitude of these differences is negligible.

The horizontal and vertical design response spectra are scaled to the maximum ground acceleration of 0.12g and 0.08g selected for the SSE. For the OBE, a scaling factor of 0.5 is applied to the SSE design spectra. The site design response spectra are applied at the various foundations of seismic Category I structures.

The specific percentage of critical damping values used in the seismic analysis of Category I structures, systems, and components is based on material, stress levels, and type of connections of the particular structure or component. These values are determined in accordance with the recommendations of Regulatory Guide 1.61 and those in Newmark's work. The synthetic time history used for the seismic design of Category I structures, systems and components is adjusted in amplitude and frequency content to obtain response spectra that enveloped the response spectra specified for the site.

3.7.2 Seismic Structural System and Subsystem Analyses

The review of the seismic system and subsystem analysis for the plant included the seismic analysis methods for all Category I structures, systems, and components, in addition to procedures for modeling, seismic soil-structure interaction, development or floor response spectra, inclusion of torsional effects, evaluation of Category I structure overturning, and determination of composite damping. The review included design criteria and procedures for evaluation of interaction of non-Category I structures and piping with Category I

^{*&}quot;Design Report Spectra for Nuclear Power Plants" presented by N. B. Newmark, J. A. Blume, and K. K. Kapur, at the ASCE Structural Engineering Meeting, San Francisco, April 1973.

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> structures and piping and the effects of parameter variations on floor response spectra. The review also included criteria and seismic analysis procedures for reactor internal and Category I buried piping outside the containment.

The system and subsystem analyses were performed by the applicant on an elastic basis. Modal response spectrum multidegree of freedom and time-history methods form the basis for the analyses of all major Category I structures, systems and components. When the modal response spectrum method is used, governing response parameters will be combined by a method that is generally more conservative than the square-root-of-the-sum-of-the-squares rule adopted as the staff position. However, the absolute sum of the modal response was used for modes with closely spaced frequencies. The square root of the sum of the squares of the maximum codirectional responses was used in accounting for three components of the earthquake motion for both the time history and response spectrum methods. Floor spectra input for design and test verification of structures, systems, and components was generated from the time-history method, taking into account variation of parameters by peak widening. Peaks were broadened ± 10% and connected without leaving valleys. When the peak broadening is less than ± 15%, the smoothing method is conservative and acceptable. A vertical seismic system dynamic analysis was employed for all structures, systems, and components where analysis showed significant structural amplification in the vertical direction. Torsional effects and stability against overturning were considered. The applicant has demonstrated to the staff that the eccentricities used in the analysis of Category I structures for the evaluation of torsional effects exceed the minimum value of ± 5% recommended by the staff. The staff finds the eccentricity values considered in the design acceptable.

The lumped-mass-spring approach is used to evaluate soil-structure interaction and structure-to-structure interaction effects and seismic responses.

For the analysis of Category I dams, a finite element approach that takes into consideration the time history of forces, the behavior and deformation of the dam caused by the earthquake, and applicable stress-strain relations is used.

The staff concludes that the seismic system and subsystem analysis procedures and criteria proposed by the applicant provide an acceptable basis for the seismic design.

3.7.3 Seismic Mechanical Subsystem Analyses

The review under SRP Section 3.7.3 included the applicant's seismic analysis of the reactor coolant system; reactor internals, core, and control rod drive mechanisms; and seismic Category I piping systems (excluding the reactor coolant system). Each of these areas is discussed below.

3.7.3.1 Reactor Coolant System

The reactor vessel, pumps, steam generators and their supports, and the interconnecting piping system were evaluated as a coupled system. The mathematical model provides a three-dimensional representation of the dynamic response of the coupled components to seismic excitations in both the horizontal and

3.7.2 Seismic Structural System and Subsystem Analyses

FSAR Figures 7.73-41 through 3.78-49, documenting response spectra, were deleted from the FSAR. In the August 16, 1989 submittal, the applicant confirmed that these spectra were not used for design of any Category I structures. However, in a letter dated January 3, 1990, the applicant committed to include sample base and top level response spectra used for the design of the CPSES Category I structures in a future FSAR revision (Amendment 78) prior to Unit 1 fuel loading. This issue is considered resolved, contingent on staff verification that the appropriate FSAR changes are made before Unit 1 fuel loading.

The applicant has agreed to revise the FSAR to document that the peaks of the floor response spectra were widened by ±10 percent rather than by only +10 percent. The applicant will also revise the FSAR to reflect that the effect of the structural backfill on the soil spring stiffness values for the service water intake structure (SWIS) was calculated based on rock and then for soil media, and that average spring stiffness was used. The staff reviewed relevant documents during the site audit on September 6-8, 1989. The results of the parametric study performed for the generation of the floor response spectra were also discussed during the audit. The parametric variation of the soil-spring stiffness had been considered in generating the original floor response spectra. The validation study considered the soil-structure interaction by modeling the soil along with the foundation. The CLASSI and FLORA computer programs were used in this validation process. The parametric variation was not considered for the new response spectra used for validation purposes. However, for the SWIS and three exterior storage tanks, new response spectra were developed considering the parametric variation. In addition, the staff verified that an average value of soil-spring stiffness between rock and soil media was used in the calculations for response spectra for the SWIS. The staff finds these approaches to be acceptable.

The FSAR did not include a discussion on the method of analysis for Category I tanks. In the meeting on July 31, 1989, the applicant agreed to revise the FSAR to provide such a discussion, including information related to the geometry of tanks at CPSES. The August 16, 1989 submittal provides the information requested by the staff. This information is also in FSAR Section 3.8.4.1.6. This information describes the method of analysis which complies with the provisions of U.S. Atomic Energy Commission Technical Information Document TID-7024, and is acceptable to the staff.

FSAR Amendment 68 stated that the structural failure of the turbine building is prevented by internal bracing. During the meeting with the staff on July 31, 1989, the applicant stated that the structural failure of the turbine building is prevented by the bearing of the mezzanine and operating floor slabs on the concrete turbine pedestal. The applicant has revised FSAR Section 3.7B.2.8 to reflect the actual support mechanism for the turbine building. During its site visit on September 6-8, 1989, the staff reviewed the assumptions and methods used in the development of the loads on the support mechanism for the turbine building, and concluded that the analysis had been performed correctly. The staff, therefore, considers this issue to be resolved.

The applicant has revised the FSAR to include missing terms and the definition of two analysis parameters in Sections 3.7N.2.1.2 and 3.7N.2.1.5. Also, since the power spectral density (PSD) function was not used to characterize the input motion, FSAR Section 3.7B.2.1.3 has been revised to delete the term FSD. In

Comanche Peak SSER 22