

ENCLOSURE 1

NOTICE OF VIOLATION

Georgia Power Company  
Hatch Units 1 and 2

Docket Nos. 50-321 and 50-366  
License Nos. DPR-57 and NPF-5

The following violations were identified during an inspection conducted on December 21, 1984 - January 28, 1985. The Severity Levels were assigned in accordance with the NRC Enforcement Policy (10 CFR Part 2, Appendix C).

1. Technical Specification 3.5.F.1.a requires the seven valves of the Automatic Depressurization System (ADS) be operable prior to startup from a cold shutdown.

Contrary to the above, one Automatic Depressurization System valve was not operable prior to startup from cold shutdown. On the "B" valve, a States Block Link was open, thereby preventing the ADS signal from reaching the solenoid which operates the "B" ADS valve. The reactor was started with this valve not operational in the manual or ADS mode.

This is a Severity Level IV Violation (Supplement I) applicable to Unit 1.

2. Technical Specification 6.8.1 requires that written procedures be established, implemented and maintained for safety-related activities.

Contrary to the above:

- a. Procedure HNP-1-6020, "Main Steam Relief Valves Maintenance", was not properly implemented in that isolation links were left open which disabled the Manual and Automatic Depressurization System mode of one safety relief valve even though the procedure was signed off as complete. Also the lifted wire and jumper tag was not removed when a link was closed.
- b. Procedure HNP-1-3902, "Safety Relief Valve Functional Test", was not implemented when required by HNP-1-6020, "Main Steam Relief Valve Maintenance", and when required as the functional test by the maintenance work order. The Unit was critical at about 150 psi when the licensee realized this procedure was not complete.
- c. Procedure HNP-1-3820, "Safety/Relief Valve Position, Primary and Secondary, Indicators Functional Test and Calibration", was not properly maintained in that the incorporation of a Design Change Request was not complete, resulting in an inadvertent initiation of Low Low Set logic and attendant actuation of a safety relief valve.

This is a Severity Level IV Violation (Supplement I) applicable to Unit 1.

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3. Technical Specification:

- a. Table 4.3.2-1, Trip Function item 1.d, requires that the Main Steam Line Tunnel Temperature-High receive a channel functional test monthly.

Contrary to the above, the monthly Main Steam Line Tunnel Temperature-High was not performed on a monthly basis. This surveillance had been an 18 month surveillance prior to Technical Specification Amendment No. 39 which changed it to monthly. Two monthly surveillances were missed.

- b. Table 4.3.6.4-1, Instrument items 1 and 2, required that the Reactor Vessel Pressure Instrumentation (2B21-R623 A&B) and the Reactor Vessel Water Level Instrumentation (2B21-R610, R615) receive a quarterly channel calibration.

Contrary to the above, due to an improper procedure change which changed the surveillance frequency from quarterly to every 18 months, the quarterly channel calibration for the Reactor Vessel Pressure Instrumentation (2B21-R623 A, B) was not performed for the period from receipt of Amendment No. 39 to Unit 2 Technical Specifications (approximately June 1984) until November 1984. The above is also the case for the Reactor Vessel Shroud Water Level Instrumentation (2BR1-R610, R615).

This is a Severity Level IV Violation (Supplement I) applicable to Unit 2.

Pursuant to 10 CFR 2.201, you are required to submit to this office within 30 days of the date of this Notice, a written statement or explanation in reply, including: (1) admission or denial of the alleged violations; (2) the reasons for the violations if admitted; (3) the corrective steps which have been taken and the results achieved; (4) corrective steps which will be taken to avoid further violations; and (5) the date when full compliance will be achieved.

Security or safeguards information should be submitted as an enclosure to facilitate withholding it from public disclosure as required by 10 CFR 2.790(d) or 10 CFR 73.21.

Date:     **FEB 25 1985**