

Northern States Power Company
ATTN: Mr. Leo Wachter, Vice President
Power Production and System
Operation
414 Nicollet Mall
Minneapolis, Minnesota 55401

Docket No. 50-263

PDR

Gentlemen:

Enclosed is IE Bulletin No. 74-10B, which amends the description of the circumstances discussed in EO Bulletin No. 74-10A and requests additional actions by you.

Should you have questions regarding this Bulletin or the actions requested of you, please contact this office.

Sincerely,

James G. Keppler
Regional Director

Enclosure:
IE Bulletin No. 74-10B

Approved by (), B-180255 (RO072), clearance expires 7/31/77.
Approval was given under a blanket clearance specifically for
identified generic problems.

bcc: DR Cent Files
RO Files
PDR
Local PDR
OGC, Beth, P-506A
A. Roisman



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January 27, 1975
IE Bulletin No. 74-10B

FAILURES IN 4-INCH BYPASS PIPING AT DRESDEN-2
(REFERENCE: RO BULLETINS NO. 74-10, DATED SEPTEMBER 19, 1974
AND NO. 74-10A, DATED DECEMBER 17, 1974)

DESCRIPTION OF CIRCUMSTANCES (AMENDED):

Cracks similar to those described in RO Bulletins 74-10 and 74-10A have been revealed during reexamination of the 4-inch bypass piping at five boiling water reactor facilities (including Dresden-2) since mid-December 1974. All the facilities are of jet-pump design. In each instance where cracks have been revealed, the same welds had been examined previously (during September-November 1974) and no cracks were revealed at that time.

A special study group, comprised of key technical members of the Nuclear Regulatory Commission staff, is currently studying the causes of the cracks which have been experienced. Consultants from other government agencies are also participating in the study.

In the interim, pending recommendations of the Commission's special study group, additional actions are necessary by licensees to insure continued close surveillance of reactor coolant leakage.

ACTIONS REQUESTED OF LICENSEES (AMENDED):

For all boiling water reactor facilities of jet-pump design with operating license:

1. Upon completion of those actions described in RO Bulletin 74-10A, implement the following reactor coolant leakage detection program:

If (within the sensitivity specified in paragraph C.5. of Regulatory Guide 1.45) any reactor coolant leakage detection system indicates, within a period of 4 hours or less, either an increase in unidentified leakage to twice the determined normal rate of unidentified leakage or an increase in the rate of unidentified leakage by two (2.0) gpm or more:

- a. Initiate plant shutdown immediately, and
- b. Determine the source of increased leakage prior to resuming operation.

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- c. If there is evidence of leakage from the 4-inch bypass piping, reexamine (by ultrasonic or other suitable volumetric inspection technique), all accessible welds in the bypass piping lines.

(In no case, however, shall the rate of leakage exceed that specified in the technical specifications without those actions required by the technical specifications being taken.)

2. Notify this office, in writing within 10 days of your receipt of this Bulletin, of your intent to implement the actions described in 1., above.