

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
Washington Nuclear Plant - Unit 2

DOCKET NUMBER (2)
050003071

PAGE (3)
1 OF 03

TITLE (4)
Surveillance Test Procedure Omission

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
04	18	85	85	032	000	06	14	85		050000
										050000

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

OPERATING MODE (9) 1	20.402(b)	20.406(a)	90.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10) 0515	20.406(a)(1)(i)	90.73(a)(1)	90.73(a)(2)(v)	73.71(e)
	20.406(a)(1)(ii)	90.73(a)(2)	90.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 368A)
	20.406(a)(1)(iii)	X 50.73(a)(2)(ii)	90.73(a)(2)(vii)(A)	
	20.406(a)(1)(iv)	90.73(a)(2)(iii)	90.73(a)(2)(vii)(B)	
	20.406(a)(1)(v)	90.73(a)(2)(iv)	90.73(a)(2)(viii)	

LICENSEE CONTACT FOR THIS LER (12)

NAME: R. L. Koenigs, Compliance Engineer

TELEPHONE NUMBER: 5101931771-1251011

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) Ext. 2279

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (if you complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 4/18/85, Instrument and Control (I&C) Department personnel were in the process of writing Technical Specification required 18 month surveillance tests and upgrading existing surveillance procedures to reflect recent equipment changes. During review of these procedures it was discovered that not all output signals from the Reactor Vessel Water Level-Low Low (Level 2) instruments were being surveilled. The output signals which were not being tested provided inputs to secondary containment isolation. These instruments also provided output signals to other safety systems which were verified during previous surveillance testing.

All surveillance test procedures that provide an input to secondary containment isolation have now been verified not to have similar deficiencies.

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (8)			PAGE (3)		
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Washington Nuclear Plant - Unit 2	0500039785	03	2	000	2	OF	03

TEXT (if more space is required, use additional NRC Form 388A's) (17)

Plant Conditions

- a) Plant Mode - 1
b) Power Level - 55%

Event

On 4/18/85, Instrument and Control (I&C) Department personnel were in the process of writing Technical Specification required 18 month surveillance tests and upgrading existing surveillance test procedures to reflect recent equipment changes. During review of these procedures it was discovered that not all output signals from the Reactor Vessel Water Level Low MS-LITS-26A, B, C and D, Level 2, instruments were being surveilled. The output signals that were not being tested provided inputs to secondary containment isolation. These same instruments also provided inputs to the primary containment isolation and Reactor Water Cleanup (RWC) System isolation which were verified during previous surveillance testing.

This discrepancy was not originally evaluated as a reportable condition. However, subsequent review by Quality Assurance and Technical Department personnel on 5/29/85 determined the event to be reportable per 50.73(a)(2)(i)(B).

During initial development of the surveillance test procedures in question, it was determined that only one set of the two sets of available contacts (both of which provided trip functions) could be adequately verified during testing. Both sets of contacts actuated on the same signal and monitoring one set was believed to be representative of the remaining contact set. Our more recent evaluation has determined that both sets of contacts can be verified during each monthly channel functional test. The Level 2 and high drywell pressure surveillance test procedures were therefore deficient in this respect from 12/20/83 through 4/19/85.

Immediate Corrective Action

The procedures involved were changed to reflect monitoring requirements for the Level 2 contacts which had not originally been included in surveillance test procedures.

Further Corrective Action

- o A review identified that drywell high pressure (RPS-PS-2A, B, C, D) output signals providing input to secondary containment isolation were not being tested. These same instruments also provide input to the Reactor Protection System (RPS) and primary containment isolation which were verified during previous surveillance testing. The procedures involved were changed to reflect monitoring requirements for the high drywell contacts which had not originally been included in surveillance test procedures. The modified procedure was then conducted and the previously un-surveilled contacts were found to be in specification. The MS-LITS-26A, B, C and D instruments were replaced with new instrumentation prior to performing the modified surveillance procedure.

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TEXT (If more space is required, use additional NRC Form 388A's) (17)

- o. All remaining procedures for instruments supplying trip inputs to secondary containment isolation were verified not to have similar deficiencies. The secondary containment isolation is unique in that all isolation signals are routed through relay cabinets RC-1 and RC-2. No further instances similar to the described event condition are believed to exist.
- o. The present system of determining reportability relies on the on-duty Shift Manager and Shift Technical Advisor for the first evaluation. The Technical Staff or Maintenance Department personnel assigned to disposition a resultant Non-Conformance Report (NCR) perform a second level evaluation. Thirdly, the Compliance Staff performs an independent review for NCR's which require further clarification or consideration. This system of identifying 10CFR50.73 Reportable Items is judged adequate in view of past performance in this area. For future events where reportability is changed subsequent to Shift Manager approval, a copy of the NCR will be returned to the Operation's Manager as a feedback mechanism. This was not previously done.

Safety Significance

Monthly channel functional testing, which verified the initiating signal and partial trip signal output, was preformed throughout the period. During a previous event, correct secondary isolation functioning had been observed. When the secondary containment isolation signal for drywell high pressure was surveilled, it was found to be in specification. It is therefore reasonable to postulate that, had any abnormal conditions been present during this event period requiring the secondary containment isolation, the Plant would have responded as designed. There was no impact to the safety of the Plant, Plant personnel or the Public as a result of this event.

Similar Events

Not Applicable

EIIS Information

EIIS Reference

Text Reference	System	Component
RPS	JC	----
Containment Isolation System	NH	----
MS-LITS-26A, B, C, D	SB	*LITS
RPS-PS-2A, B, C, D	JC	PS
RWCU	CE	----

Washington Public Power Supply System

P.O. Box 968 3000 George Washington Way Richland, Washington 99352 (509) 372-5000

Docket No. 50-397

June 14, 1985

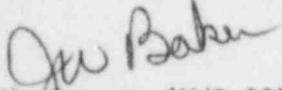
Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: NUCLEAR PLANT NO. 2
LICENSEE EVENT REPORT NO. 85-032

Dear Sir:

Transmitted herewith is Licensee Event Report No. 85-J32 for WNP-2 Plant. This report is submitted in response to the report requirements of 10CFR50.73 and discusses the item of reportability, corrective action taken, and action taken to preclude recurrence.

Very truly yours,


C.M. Powers (M/D 927M)
WNP-2 Plant Manager

CMP:1a

Enclosure:
Licensee Event Report No. 85-032

cc: Mr. John B. Martin, NRC - Region V
Mr. A. D. Toth, NRC - Site (901A)
Ms. Dottie Sherman, ANI
INPO Records Center - Atlanta, GA

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