

**Commonwealth Edison** One First National Plaza, Chicago, Illinois Address Reply to: Post Office Box 767 Chicago, Illinois 60690

May 2, 1985

Mr. James G. Keppler Regional Administrator U. S. Nuclear Regulatory Commission Region III 799 Roosevelt Road Glen Ellyn, IL 60137

> Subject: Braidwood Station Units 1 and 2 10 CFR 50.55(e) No. 84-10 Interim Report Piping Wall Thickness Deficiencies NRC Docket Nos. 50-456/457

- References (a): E. D. Swartz letter to J. G. Keppler dated July 20, 1984
  - (b): D. H. Smith letter to J. G. Keppler dated September 18, 1984
  - (c): D. H. Smith letter to J. G. Keppler dated November 2, 1984
  - (d): D. H. Smith letter to J. G. Keppler dated December 13, 1984
  - (e): D. H. Smith letter to J. G. Keppler dated February 27, 1985

Dear Mr. Keppler:

References (a) thru (e) provided information concerning a deficiency reportable pursuant to 10 CFR 50.55(e) regarding wall thickness inadequacies for small bore ASME Class II piping at our Braidwood Station. The purpose of this letter is to provide an updated status of the corrective actions taken to resolve this issue. This letter is considered to be an Interim Report.

## CORRECTIVE ACTION STATUS

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As previously reported, Commonwealth Edison Company NCR 633 (Rev. 1) was written to track issue resolution. All suspect pipe in storage has been placed on hold pending the dispositioning of installed pipe.

A corrective action program was developed by Commonwealth Edison Company and Sargent & Lundy to disposition the subject pipe that has been installed. The corrective action program includes the following elements:

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- Laboratory testing to verify the design significant properties of the suspect pipe.
- Analysis to evaluate the effect of prolonged outdoor storage and subsequent chemical cleaning.
- Measurement of the actual wall thickness of samples of suspect pipe taken from both installed pipe and pipe on hold in storage.
- Analysis of laboratory testing results and pipe wall thickness measurements to determine the acceptability of installed piping relative to ASME Section III design requirements.

Results of the laboratory tests on the suspect pipe are currently being evaluated by Sargent & Lundy for design significance.

An inventory of suspect pipe installed in the plant and on hold in storage has been completed. Based on a total of approximately 28,700 pieces, a random sample of 300 pieces has been selected for detailed measurement. This sample size was selected to satisfy a 95% confidence/99% reliability criterion. Work is presently underway, in accordance with Phillips Getschow Co. Procedure PGCP-52, to measure the wall thickness of this sample. Current estimates indicate the sample measurements are expected to be completed by July 1, 1985.

A final report, covering the analysis of laboratory testing, pipe wall measurements, and effects of prolonged outdoor storage and subsequent chemical cleaning is expected to be completed by August, 1985.

Please address any questions that you or your staff may have concerning this matter to this office.

Very truly yours,

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Anthony D. Miosi Nuclear Licensing Administrator

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cc: NRC Resident Inspector - Braidwood

Director of Inspection and Enforcement U.S. Nuclear Regulatory Commission Washington, D.C. 20555