

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) JAMES A. FITZPATRICK NUCLEAR POWER PLANT	DOCKET NUMBER (2) 0 5 0 0 0 3 3 3	PAGE (3) 1 OF 0 2
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TITLE (4)
Generic Setpoint Drift

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)			
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES			DOCKET NUMBER(S)
0 5	1 1	8 5	8 5	0 1 4	0 0 0	6 1 0	8 5					0 5 0 0 0
												0 5 0 0 0

OPERATING MODE (9) N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)									
POWER LEVEL (10) 0 0 0	<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.406(c)	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)						
	<input type="checkbox"/> 20.406(a)(1)(i)	<input checked="" type="checkbox"/> 50.36(c)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(a)						
	<input type="checkbox"/> 20.406(a)(1)(ii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)						
	<input type="checkbox"/> 20.406(a)(1)(iii)	<input type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)							
	<input type="checkbox"/> 20.406(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)							
	<input type="checkbox"/> 20.406(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(x)							

LICENSEE CONTACT FOR THIS LER (12)

NAME Hartford N. Keith	TELEPHONE NUMBER
	AREA CODE 3 1 5
	3 4 2 - 3 8 4 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)										
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input type="checkbox"/> NO			

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

With the plant shutdown for refueling and while performing a functional/calibration surveillance test Reactor High Pressure LC Switch, 02 PS 128A was found to actuate at 81 psig. The operating Technical Specification Table 3.2-2 value is 50 < P < 75 psig. The redundant switch 02 PS 128 B was found to actuate at 82 psig.

These switches serve as a permissive for opening the LPCI valves 10MOV25 A & B when the shutdown cooling mode is initiated provided a LOCA signal is not present and reactor pressure is < 450 psig.

The switches were immediately recalibrated and tested successful per the surveillance procedure. An increased surveillance frequency of once per week was also established for trend observation. Four (4) surveillance have been completed since the May 11, 1985, occurrence and no drift outside the established band of 62 (54 - 70) psig has been observed. Due to this satisfactory behavior, switches will be returned to the required monthly functional surveillance frequency.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		8 5	- 0 1 4	- 0 0	0 2	OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

With the plant shutdown for refueling and while performing a required before plant startup functional/calibration surveillance test, Reactor High Pressure switch 02 PS 128A was found to actuate at 81 psig. The operating Technical Specifications Table 3.2-2 requirement is actuation less than or equal to 75 psig. The redundant switch 02 PS 128 B was found to actuate at 82 psig.

These switches serve the reactor shutdown cooling subsystem which serve to remove reactor stored/decay heat during normal shutdown operation and after reduction of reactor vessel pressure. They are an integrated part of the reactor core residual heat removal (RHR) system.

The reactor shutdown cooling subsystem suction line is provided with two isolation valves at the containment penetration which in conjunction with switches 02 PS 128A serve as a permissive for opening the RHR Low Pressure Coolant Injection (LPCI) injection valves 10 MOV 25 A & B. Provided either reactor low water level or high drywell pressure or high reactor pressure above the LPCI system design pressure signal are not present.

This combination of pressure and valve interlocks prevents inadvertent reactor loss of coolant through the cooling systems lines during the reactor shutdown cooling operation. It also ensures valves are properly positioned for LPCI system operations as needed, thus permitting use of common pumps, heat exchangers, and valves for LPCI and shutdown cooling operation.

The switches were immediately recalibrated and tested successfully per the surveillance procedure. An increased surveillance frequency of once per week was also established for trend observations. Four (4) tests have been completed since the May 11, 1985, occurrence and no drift outside the established band of 62 (54 -70) psig has been observed. Due to this satisfactory behavior switches will be returned to the required monthly functional surveillance frequency.

James A. FitzPatrick
Nuclear Power Plant
P.O. Box 41
Lycoming, New York 13093
315 342.3840

Memorandum



New York Power Authority

June 11, 1985
JAFF85-0501

United States Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

REFERENCE: DOCKET NO. 50-333
LICENSEE EVENT REPORT: 85-014

Dear Sir:

Enclosed please find the referenced Licensee Event Report in accordance with the requirements of 10 CFR 50.73.

If there are any questions concerning this report, please contact Mr. Hartford N. Keith at (315) 342-3840, Extension 230.

A handwritten signature in cursive script, appearing to read 'H. A. Glovier'.

HAROLD A. GLOVIER

HAG/HNK/cmd
Enclosure

CC: USNRC, Region I (1)
INPO Records Center, Atlanta, Georgia (1)
Internal Power Authority Distribution
NRC Resident Inspector
Document Control Center
LER/OR File

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