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January 15, 1993

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D.C. 20555

Gentlemen:

Subject: Docket Nos. 50-361 and 50-362
Monthly Operating Reports for December 1992
San Onofre Nuclear Generating Station, Units 2 and 3

Technical Specification 6.9.1.10 to Facility Operating Licenses NPF-10 and NPF-15 for the San Onofre Nuclear Generating Station, Units 2 and 3, respectively, requires SCE provide a Monthly Operating Report for each Unit, which includes: routine operating statistics and shutdown experience; all challenges to safety valves; any changes to the Offsite Dose Calculation Manual (ODCM); and any major changes to the radioactive waste treatment system. All covered activities are reported monthly, except for ODCM changes, which are reported within 90 days from the time the changes were made effective.

This letter transmits the December 1992 Monthly Operating Reports for Units 2 and 3, respectively. There were no challenges to safety valves, no changes to the ODCM, and no major changes to the Units 2 and 3 radioactive waste treatment systems during the reporting period.

If you require any additional information, please let me know.

Very truly yours,

Ron Baenkh

Enclosures

cc: J. B. Martin (Regional Administrator, USNRC Region V)
M. B. Fields (NRC Project Manager, Units 2 and 3)
C. W. Caldwell (USNRC Senior Resident Inspector, Units 1, 2 and 3)

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NRC MONTHLY OPERATING REPORT

DOCKET NO: 50-361
UNIT NAME: SONGS - 2
DATE:
COMPLETED BY: J. L. Darling
TELEPHONE: (714) 368-6223

OPERATING STATUS

1. Unit Name: San Onofre Nuclear Generating Station, Unit 2
2. Reporting Period: December 1992
3. Licensed Thermal Power (Mwt): 3390
4. Nameplate Rating (Gross Mwe): 1127
5. Design Electrical Rating (Net Mwe): 1070
6. Maximum Dependable Capacity (Gross Mwe): 1127
7. Maximum Dependable Capacity (Net MWe): 1070
8. If Changes Occur In Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons: NA
9. Power Level To Which Restricted, If Any (Net MWe): NA
10. Reasons For Restrictions, If Any: NA

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744.00	8,784.00	82,177.00
12. Number Of Hours Reactor Was Critical	744.00	8,242.11	60,734.36
13. Reactor Reserve Shutdown Hours	0.00	0.00	0.00
14. Hours Generator On-Line	744.00	8,215.72	59,657.27
15. Unit Reserve Shutdown Hours	0.00	0.00	0.00
16. Gross Thermal Energy Generated (MWH)	2,449,119.61	27,297,812.89	194,819,848.32
17. Gross Electrical Energy Generated (MWH)	830,728.50	9,244,257.50	66,045,560.00
18. Net Electrical Energy Generated (MWH)	791,594.00	8,795,181.00	62,607,515.83
19. Unit Service Factor	100.00%	93.53%	72.60%
20. Unit Availability Factor	100.00%	93.53%	72.60%
21. Unit Capacity Factor (Using MDC Net)	99.44%	93.58%	71.20%
22. Unit Capacity Factor (Using DER Net)	99.44%	93.58%	71.20%
23. Unit Forced Outage Rate	0.00%	6.47%	6.87%

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): Refueling shutdown, May 1993 to August 1993 (77 days)

25. If Shutdown At End Of Report Period, Estimated Date of Startup: NA

26. Units In Test Status (Prior To Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	NA	NA
INITIAL ELECTRICITY	NA	NA
COMMERCIAL OPERATION	NA	NA

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 50-361
UNIT NAME: SONGS - 2
DATE:
COMPLETED BY: J. L. Darling
TELEPHONE: (714) 368-6223

MONTH: December 1992

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	1093.92	16	1099.08
2	1094.79	17	1094.96
3	1095.04	18	1096.25
4	1061.04	19	1093.42
5	810.83	20	1100.33
6	702.88	21	1097.96
7	1032.75	22	1097.33
8	1088.71	23	1097.92
9	1097.21	24	1100.00
10	1088.08	25	1085.04
11	1057.75	26	1085.13
12	1032.54	27	1098.88
13	1043.33	28	1097.25
14	1052.08	29	1096.75
15	1098.25	30	1097.21
		31	1096.38

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO: 50-361
UNIT NAME: SONGS - 2
DATE:
REPORT MONTH: December 1992
COMPLETED BY: J. L. Darling
TELEPHONE: (714) 368-6223

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	LER No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
79	921205	S	0.0	B	NA	NA	KE	COND	Load reduced to 80% to perform circulating water heat treat and other reduced load maintenance. After the heat treat was completed load was further reduced to 60% because one Control Element Assembly slipped into the core during monthly operability testing. The cause of slipping was identified as a loose connector for the lower gripper. The connector was properly reconnected and the CEA was aligned with its bank.

[1]F-Forced
S-Scheduled

[2]Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

[3]Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuation from Previous Month
5-Reduction in the Average Daily Power Level of more than 20% from the previous day
6-Other (Explain)

[4]IEEE Std 805-1984

[5]IEEE Std 803A-1983

SUMMARY OF OPERATING EXPERIENCE FOR THE MONTH

DOCKET NO: 50-361
UNIT NAME: SONGS - 2
DATE:
COMPLETED BY: J. L. Darling
TELEPHONE: (714) 368-6223

Date	Time	Event
December 1	0001	Unit is in Mode 1, 100% Reactor power, 1141 MWe.
December 4	1930	Commenced load reduction to 80% power for circulating water heat treat.
December 5	0001	Unit at 81% Reactor power.
	1315	Commenced load reduction to 75% power to support feedwater pump and circulating water pump maintenance following completion of heat treat.
	1400	Unit at 75% Reactor power
December 6	1630	Control Element Assembly (CEA) #85 slipped from 145 to 93 inches. Reactor and turbine load stable.
	1640	CEA #85 slipped to 54 inches during realignment attempt.
	1645	Commenced load reduction to 60% Reactor power per Tech. Spec. 3.1.3.1
	1657	CEA #85 Slipped fully into the core during attempted withdrawal.
	1700	Reactor power less than 60%.
	1910	Commenced withdrawing CEA #85 after the cause of slippage was identified and corrected.
	2015	CEA #85 properly aligned.
	2130	Commenced load increase.
December 7	0725	Unit at 99.8% reactor power.
December 11	0745	Commenced power reduction and temperature decrease because of DNBR pre-trip alarm. Isothermal temperature coefficient (ITC) testing caused a negative Axial Shape Index.

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SUMMARY OF OPERATING EXPERIENCE FOR THE MONTH

DOCKET NO: 50-361
UNIT NAME: SONGS - 2
DATE:
COMPLETED BY: J. L. Darling
TELEPHONE: (714) 368-6223

December 11	1315	Reactor power at 95% for ITC testing
December 14	1920	Commenced raising Reactor power to 100%
	2132	Unit at 100% Reactor power.
December 31	2400	Unit is in Mode 1, 100% reactor power, 1148 MWe.

mor.dec/6

REFUELING INFORMATION

DOCKET NO: 50-361
UNIT NAME: SONGS - 2
DATE:
COMPLETED BY: J. L. Darling
TELEPHONE: (714) 368-6223

MONTH: December 1992

1. Scheduled date for next refueling shutdown.

   Cycle 7 refueling outage is forecast for May 1993.

2. Scheduled date for restart following refueling.

   Restart from Cycle 7 refueling outage is forecast for August 1993.

3. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment?

   Yes.

   What will these be?

   The following Technical Specification changes are desired in support of work being performed during the Unit 2 Cycle 7 refueling outage.

   A. A change to Technical Specification 3.9.7 has been requested to permit use of the cask pool cover to support refueling activities. This cover had previously been used to support the spent fuel pool reracking project. NRC approval of this change will be required prior to the start of the outage to support use of the cover during the outage.

   B. A change has been requested to Technical Specification 4.4.5.2.1 to allow a delay in performing an RCS inventory balance during plant transients. Although not required, NRC approval of this change is desired prior to startup from the Unit 2 Cycle 7 outage.

   C. A change will be requested to Technical Specification 3.7.1.1 to allow a tolerance on the main steam safety valves for the purpose of determining valve operability. Although not required, NRC approval of this change is desired to support the surveillance testing scheduled to be performed during the Unit 2 Cycle 7 outage.

mor.dec/7

REFUELING INFORMATION

DOCKET NO: 50-361
UNIT NAME: SONGS - 2
DATE:
COMPLETED BY: J. L. Darling
TELEPHONE: (714) 368-6223

MONTH: December 1992

D. A license Amendment and Technical Specification changes has been requested to permit implementation of a design change to use the containment spray pumps for shutdown cooling and spent fuel pool cooling. NRC approval of this change will be requested to permit use of the spray pumps for spent fuel pool cooling during the Unit 2 cycle 7 outage.

4. Scheduled date for submitting proposed licensing action and supporting information.

A.	Proposed Change on Cask Pool Cover	Submitted December 24, 1992
B.	Proposed Change on RCS Leakrate	Submitted November 20, 1992
C.	Proposed Change on MSSVs	January 31, 1992
D.	Proposed Change on Spray Pumps	Submitted December 24, 1992

5. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

None.

6. The number of fuel assemblies.

a) In the core. 217

b) In the spent fuel storage pool. 554 (484 Unit 2 Spent Fuel Assemblies, 70 Unit 1 Spent Fuel Assemblies)

7. Licensed spent fuel storage capacity. 1542

Intended change in spent fuel storage capacity. None

8. Projected date of last refueling that can be discharged to spent fuel storage pool assuming present capacity.

Approximately 2001 (full off load capability)

mor.dec/8

NRC MONTHLY OPERATING REPORT

DOCKET NO: 50-362
UNIT NAME: SONGS - 3
DATE:
COMPLETED BY: J. L. Darling
TELEPHONE: (714) 368-6223

OPERATING STATUS

1. Unit Name: San Onofre Nuclear Generating Station, Unit 3
2. Reporting Period: December 1992
3. Licensed Thermal Power (MWt): 3390
4. Nameplate Rating (Gross MWe): 1127
5. Design Electrical Rating (Net MWe): 1080
6. Maximum Dependable Capacity (Gross MWe): 1127
7. Maximum Dependable Capacity (Net MWe): 1080
8. If Changes Occur In Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons: NA
9. Power Level To Which Restricted, If Any (Net MWe): NA
10. Reasons For Restrictions, If Any: NA

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744.00	8,784.00	76,728.00
12. Number Of Hours Reactor Was Critical	744.00	6,701.51	59,199.76
13. Reactor Reserve Shutdown Hours	0.00	0.00	0.00
14. Hours Generator On-Line	744.00	6,535.05	57,606.06
15. Unit Reserve Shutdown Hours	0.00	0.00	0.00
16. Gross Thermal Energy Generated (MWH)	2,508,783.20	21,346,649.78	184,634,074.24
17. Gross Electrical Energy Generated (MWH)	865,492.50	7,226,835.00	62,626,860.00
18. Net Electrical Energy Generated (MWH)	822,807.00	6,827,563.07	59,145,670.36
19. Unit Service Factor	100.00%	74.40%	75.08%
20. Unit Availability Factor	100.00%	74.40%	75.08%
21. Unit Capacity Factor (Using MDC Net)	102.40%	71.97%	71.37%
22. Unit Capacity Factor (Using DER Net)	102.40%	71.97%	71.37%
23. Unit Forced Outage Rate	0.00%	5.15%	7.46%

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): None
25. If Shutdown At End Of Report Period, Estimated Date of Startup: NA
26. Units In Test Status (Prior To Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	NA	NA
INITIAL ELECTRICITY	NA	NA
COMMERCIAL OPERATION	NA	NA

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 50-361
UNIT NAME: SONGS - 3
DATE:
COMPLETED BY: J. L. Darling
TELEPHONE: (714) 368-6223

MONTH: December 1992

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	1102.92	16	1111.42
2	1103.58	17	1111.83
3	1103.00	18	1108.96
4	1100.42	19	1105.71
5	1087.38	20	1112.08
6	1099.21	21	1109.96
7	1101.50	22	1109.58
8	1101.71	23	1109.83
9	1102.83	24	1109.83
10	1106.00	25	1088.42
11	1101.92	26	1098.42
12	1102.67	27	1111.13
13	1110.08	28	1110.79
14	1111.71	29	1111.33
15	1111.46	30	1114.58
		31	1113.38

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UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH: December 1992

DOCKET NO: 50-362
UNIT NAME: SONGS - 3
DATE:
COMPLETED BY: J. L. Darling
TELEPHONE: (714) 368-6223

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	LER No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence

There were no unit shutdowns or load reductions this reporting period.

[1]F-Forced
S-Scheduled

[2]Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

[3]Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuation from Previous Month
5-Reduction in the Average Daily Power Level of more than 20% from the previous day
6-Other (Explain)

[4]IEEE Std 805-1984

[5]IEEE Std 803A-1983

mor.dec/11

SUMMARY OF OPERATING EXPERIENCE FOR THE MONTH

DOCKET NO: 50-362
UNIT NAME: SONGS - 3
DATE:
COMPLETED BY: J. L. Darling
TELEPHONE: (714) 368-6223

Date	Time	Event
December 1	0001	Unit is in Mode 1 at 99.1% reactor power, 1148 MWe.
December 31	2400	Unit is in Mode 1, 100% reactor power, 1159 MWe.

REFUELING INFORMATION

DOCKET NO: 50-362
UNIT NAME: SONGS - 3
DATE:
COMPLETED BY: J. L. Darling
TELEPHONE: (714) 368-6223

MONTH: December 1992

1. Scheduled date for next refueling shutdown.

   Cycle 7 refueling outage is forecast for September 1993.

2. Scheduled date for restart following refueling.

   Restart from Cycle 7 refueling outage is forecast for December 1993.

3. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment?

   Yes.

   What will these be?

   The following Technical Specification changes are desired in support of work being performed during the Unit 3 Cycle 7 refueling outage.

   A. A change to Technical Specification 3.9.7 has been requested to permit use of the cask pool cover to support refueling activities. This cover had previously been used to support the spent fuel pool reracking project. NRC approval of this change will be required prior to the start of the outage to support use of the cover during the outage.

   B. A change has been requested to Technical Specification 4.4.5.2.1 to allow a delay in performing an RCS inventory balance during plant transients. Although not required, NRC approval of this change is desired prior to startup from the Unit 3 Cycle 7 outage.

   C. A change will be requested to Technical Specification 3.7.1.1 to allow a tolerance on the main steam safety valves for the purpose of determining valve operability. Although not required, NRC approval of this change is desired to support the surveillance testing scheduled to be performed during the Unit 3 Cycle 7 outage.

mor.dec/13

REFUELING INFORMATION

DOCKET NO: 50-362
UNIT NAME: SONGS - 3
DATE:
COMPLETED BY: J. L. Darling
TELEPHONE: (714) 368-6223

MONTH: December 1992

D. A license Amendment and Technical Specification changes has been requested to permit implementation of a design change to use the containment spray pumps for shutdown cooling and spent fuel pool cooling. NRC approval of this change will be requested to permit use of the spray pumps for spent fuel pool cooling during the Unit 3 cycle 7 outage.

4. Scheduled date for submitting proposed licensing action and supporting information.

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B.	Proposed Change on RCS Leakrate	Submitted November 20, 1992
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5. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, signif.cant changes in fuel design, new operating procedures.

None.

6. The number of fuel assemblies.

a) In the core. 217

b) In the spent fuel storage pool. 553 (484 Unit 3 Spent Fuel Assemblies, 69 Unit 1 Spent Fuel Assemblies)

7. Licensed spent fuel storage capacity. 1542

Intended change in spent fuel storage capacity. None

8. Projected date of last refueling that can be discharged to spent fuel storage pool assuming present capacity.

Approximately 2003 (full off load capability)

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