

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) EDWIN I. HATCH, UNIT II	DOCKET NUMBER (2) 0 5 0 0 0 0 3 6 6	PAGE (3) 1 OF 0 2
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TITLE (4)
FAILURE OF RCIC's OVERSPEED TRIP DEVICE

EVENT DATE (5)			LER NUMBER (8)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0	5	8	5	024	00	0	6	1			0 5 0 0 0
0	5	1	8	5		0	6	1			0 5 0 0 0

OPERATING MODE (9) 3	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)						
POWER LEVEL (10) 0 0 0	20.402(b)	20.406(c)	50.73(a)(2)(iv)	73.71(b)			
	20.406(a)(1)(i)	50.36(c)(1)	<input checked="" type="checkbox"/> 50.73(a)(2)(v)	73.71(c)			
	20.406(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)			
	20.406(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)				
	20.406(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(vii)(B)				
20.406(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(ix)					

LICENSEE CONTACT FOR THIS LER (12)		TELEPHONE NUMBER	
NAME	AREA CODE	NUMBER	
Steven B. Tipps, Superintendent of Regulatory Compliance	9 1 2	3 6 7 + 7 8 5 1	

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)										
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	
X	BN	MON	X 9 19 9	Y						
X	BN	CL	G D 8 10	Y						

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO	MONTH	DAY YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 05/16/85, with the reactor mode switch in the Startup and Hot Standby position, and reactor power at approximately 6 MWt (less than 1% power), and during performance of the "RCIC ELECTRICAL OVERSPEED MONITOR CALIBRATION" procedure (HNP-2-5299), plant personnel noted that the RCIC electrical overspeed monitor (2E51-N758) would not trip the trip and throttle valve.

On 05/18/85, with the reactor mode switch in the startup and Hot Standby position, and reactor power at approximately 10 MWt (less than 1% power), and during performance of the "RCIC PUMP OPERABILITY" procedure (HNP-2-3405), plant personnel noted that the RCIC trip and throttle valve would not trip manually.

These events were the result of component failure.

The failed components were replaced. RCIC was functionally tested satisfactorily and returned to service on 05/23/85.

There is no known previous similar event.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) EDWIN I. HATCH, UNIT II	DOCKET NUMBER (2) 0 5 0 0 0 3 6 6 8 5	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		8 5	- 0 2 4	- 0 0 0	2	OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

This 30-day LER is required by 10 CFR50.73 (a)(2)(v).

On 05/16/85, at approximately 2300 CDT with the reactor mode switch in the Startup and Hot Standby position, and reactor power at approximately 6 Mwt (less than 1% power), and during performance of the "RCIC ELECTRICAL OVERSPEED MONITOR CALIBRATION" procedure (HNP-2-5299), plant personnel noted that the RCIC electrical overspeed monitor (2E51-N758) would not trip the trip and throttle valve. Reactor pressure was less than 150 psig, thus the requirements of Tech. Specs. section 3.7.3, ACTION did not apply.

On 05/18/85, at approximately 0130 CDT with the reactor mode switch in the startup and Hot Standby position, and reactor power at approximately 10 Mwt (less than 1% power), and during performance of the "RCIC PUMP OPERABILITY" procedure (HNP-2-3405), plant personnel noted that the RCIC trip and throttle valve would not trip manually. Since reactor pressure was at approximately 155 psig, plant personnel demonstrated HPCI operable per the requirements of Tech. Specs. section 3.7.3, ACTION. Thus, the event had no adverse effect on plant safety.

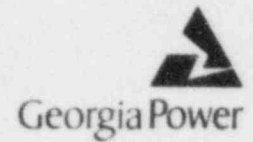
An investigation revealed that the first event was the result of the failure of the monitor's electronic circuit board. The second event was the result of the RCIC turbine's trip solenoid's coils having open windings.

Corrective action for both events was as follow:

1. On 05/17/85, plant personnel replaced the RCIC overspeed monitor.
2. On 05/18/85, plant personnel replaced the solenoid's trip coils.
3. Both the overspeed monitor and the solenoid's trip coils were functionally tested satisfactorily per HNP-2-3405 and RCIC was returned to service on 05/23/85 at approximately 1115 CDT.

These events had no impact on any other Unit 2 system or on Unit 1. There is no known previous similar event.

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Edwin I. Hatch Nuclear Plant

June 12, 1985

GM-85-582

PLANT E. I. HATCH
Licensee Event Report
Docket No. 50-366

United States Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Attached is Licensee Event Report No. 50-366/1985-024. This report is required by 10CFR50.73(a)(2)(v).

CT Jones for

H. C. Nix
General Manager

SB1
HCN/SBT/vlz

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