



GULF STATES UTILITIES COMPANY

RIVER BEND STATION POST OFFICE BOX 220 ST. FRANCISVILLE, LOUISIANA 70775
AREA CODE 504 635-9094 348-8851

January 8, 1993

RBG- 37973

File Nos. G9.5, G9.25.1.3

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Gentlemen:

River Bend Station - Unit 1
Docket No. 50-458

Please find enclosed Revision 1 to Licensee Event Report No. 92-027 for River Bend Station -Unit 1. This revision is submitted to clarify this report.

Sincerely,

W. H. Odell
Manager - Oversight
River Bend Nuclear Group

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cc: U.S. Nuclear Regulatory Commission
611 Ryan Plaza Drive, Suite 400
Arlington, TX 76011

NRC Resident Inspector
P.O. Box 1051
St. Francisville, LA 70775

INPO Records Center
1100 Circle 75 Parkway
Atlanta, GA 30339-3064

Mr. C.R. Oberg
Public Utility Commission of Texas
7800 Shoal Creek Blvd., Suite 400 North
Austin, TX 78757

Department of Environmental Quality
Radiation Protection Division
P.O. Box 82135
Baton Rouge, LA 70884-2135
ATTN: Administrator

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 500 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNRB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) RIVER BEND STATION	DOCKET NUMBER (2) 05000458	PAGE (3) 1 OF 3
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TITLE (4) **VIOLATION OF TECHNICAL SPECIFICATION 3.0.4 DUE TO FAILURE TO PLACE THE REACTOR CORE ISOLATION COOLING SYSTEM IN THE STANDBY LINEUP**

EVENT DATE (5)			LER NUMBER (6)			REPORT NUMBER (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
11	25	92	92	027	01	01	08	93		05000
									FACILITY NAME	DOCKET NUMBER
										05000

OPERATING MODE (9)	2	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)								
POWER LEVEL (10)	0	20.402(b)			20.405(c)			50.73(a)(2)(iv)		73.71(b)
		20.405(a)(1)(i)			50.36(c)(1)			50.73(a)(2)(v)		73.71(c)
		20.405(a)(1)(ii)			50.36(c)(2)			50.73(a)(2)(vii)		OTHER
		20.405(a)(1)(iii)			X 50.73(a)(2)(i)			50.73(a)(2)(vii)(A)		Specify in Abstract below and in Text, NRC Form 366A
		20.405(a)(1)(iv)			50.73(a)(2)(ii)			50.73(a)(2)(vii)(B)		
20.405(a)(1)(v)			50.73(a)(2)(iii)			50.73(a)(2)(x)				

LICENSEE CONTACT FOR THIS LER (12)

NAME L.A. ENGLAND, DIRECTOR - NUCLEAR LICENSING	TELEPHONE NUMBER (Include Area Code) (504) 381-4145
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRRDS

SUPPLEMENTAL REPORT EXPECTED (14)				EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES <small>(If yes, complete EXPECTED SUBMISSION DATE)</small>	X NO						

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On 11/25/92 at approximately 2059 hours, with the unit in Operational Condition 2 (Startup), the reactor core isolation cooling (RCIC) system was not placed in the standby lineup prior to reactor vessel pressure increasing to above 150 psig. This action is prohibited by Technical Specification 3.0.4. Therefore, this report is submitted pursuant to 10 CFR 50.73(a)(2)(i)(B) as operation prohibited by the plant Technical Specifications.

The root cause of this event was (1) incorrect interpretation of Technical Specification 3.7.3 by the Shift Supervisor and the Administrative Control Operating Foreman and (2) the failure of the Control Operating Foreman to challenge the Shift Supervisor on not placing RCIC in standby lineup, contrary to the requirements he read in general operating procedure (GOP)-0001 (Reactor Startup).

REQUIRED NUMBER OF DIGITS/CHARACTERS
FOR EACH BLOCK

BLOCK NUMBER	NUMBER OF DIGITS/CHARACTERS	TITLE
1	UP TO 46	FACILITY NAME
2	8 TOTAL 3 IN ADDITION TO 05000	DOCKET NUMBER
3	VARIES	PAGE NUMBER
4	UP TO 76	TITLE
5	6 TOTAL 2 PER BLOCK	EVENT DATE
6	7 TOTAL 2 FOR YEAR 3 FOR SEQUENTIAL NUMBER 2 FOR REVISION NUMBER	LER NUMBER
7	6 TOTAL 2 PER BLOCK	REPORT DATE
8	UP TO 18 -- FACILITY NAME 8 TOTAL -- DOCKET NUMBER 3 IN ADDITION TO 05000	OTHER FACILITIES INVOLVED
9	1	OPERATING MODE
10	3	POWER LEVEL
11	1 CHECK BOX THAT APPLIES	REQUIREMENTS OF 10 CFR
12	UP TO 50 FOR NAME 14 FOR TELEPHONE	LICENSEE CONTACT
13	CAUSE VARIES 2 FOR SYSTEM 4 FOR COMPONENT 4 FOR MANUFACTURER NPRDS VARIES	EACH COMPONENT FAILURE
14	1 CHECK BOX THAT APPLIES	SUPPLEMENTAL REPORT EXPECTED
15	6 TOTAL 2 PER BLOCK	EXPECTED SUBMISSION DATE

**LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION**

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 60.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNRB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20585-0001, AND TO THE PAPERWORK REDUCTION PROJECT (5150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (4)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
RIVER BEND STATION	05000 458	92	027	01	2 OF 3

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

REPORTED CONDITION

On 11/25/92 at approximately 2059 hours, with the unit in Operational Condition 2 (Startup), the reactor core isolation cooling (RCIC) system was not placed in the standby lineup prior to reactor vessel pressure increasing to above 150 psig. This action is prohibited by Technical Specification (TS) 3.0.4. Therefore, this report is submitted pursuant to 10 CFR 50.73(a)(2)(i)(B) as operation prohibited by the plant Technical Specifications.

INVESTIGATION

Prior to the event, reactor startup was in progress. The RCIC system had been isolated on the previous shift, prior to automatic isolation on low steam supply pressure. Criticality was achieved at 1945 on 11/25/92. Shortly following entry into the heat up range on the IRM's, pressure started to increase. At this point, a warm up of RCIC was initiated per system operating procedure (SOP)-0035. The Shift Supervisor and Administrative Control Operating Foreman (Admin. COF) reviewed Technical Specification 3.7.3 for RCIC. It was determined that when 150 psig reactor vessel pressure was reached, a limiting condition for operation (LCO) would be written. This would require the high pressure core spray system (HPCS) to be operable, and it was. The Shift Supervisor directed the Control Operating Foreman to inform the Admin. COF when 150 psig was obtained so that the LCO could be initiated. At this time, the Shift Supervisor and Admin. COF did not realize that going above 150 psig reactor vessel pressure was a change in the specified applicability conditions of TS 3.7.3. There was some confusion concerning the footnote in TS 3.7.3 that allowed going above 150 psig for RCIC surveillance testing.

The COF was following GOP-0001 for the reactor startup. The COF had read the step requiring RCIC to be in the standby mode prior to exceeding 150 psig reactor vessel pressure, but did not challenge the directions given by the Shift Supervisor for initiation of an LCO at 150 psig reactor vessel pressure. Upon re-evaluating his decision, the Shift Supervisor contacted the acting Assistant Operations Supervisor and Licensing personnel. It was determined that RCIC was required to be in the standby mode prior to exceeding 150 psig RPV pressure. The RCIC system was placed in standby lineup at 2158 hours on 11/25/92.

ROOT CAUSE

The root cause of this event was twofold. First, Technical Specification 3.7.3 was incorrectly interpreted by the Shift Supervisor and the Admin. COF. Second, the COF failed to challenge the Shift Supervisor on not placing RCIC in standby lineup, contrary to the requirements that he read in

**LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION**

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THE INFORMATION COLLECTION REQUEST: 300 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNRB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (4)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
RIVER BEND STATION	05000 458	92	- 027 -	01	3 OF 3

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

GOP-0001.

CORRECTIVE ACTION

The corrective action contains 5 requirements:

1. Training will be initiated for all ROs and SROs on interpretation of TS 3.7.3 and the footnote.
2. A change notice will be initiated for GOP-0001, for enhancement, that adds a CAUTION to the step that initiates warming up of RCIC.
3. The need for disciplinary action will be evaluated and administered by plant management, as appropriate.
4. A human performance enhancement evaluation system (HPES) review will be performed for this event to develop a case study with the Operations Department.
5. The case study will be analyzed by each operating crew by 5/30/93.

SAFETY ASSESSMENT

During this event, the low pressure emergency core cooling systems (ECCS), low pressure coolant injection (LPCI) A, B, and C and the low pressure core spray (LPCS) system were operable. Also the high pressure ECCS system, the high pressure core spray system (HPCS), was operable. Adequate core cooling was assured during the entire event.