

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Wolf Creek Generating Station	0 5 0 0 0 4 8 2	1 OF 0 2

TITLE (4)

ESF Actuation - Feedwater Isolation

EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																								
MONTH	DAY	YEAR	YEAR		SEQUENTIAL NUMBER		REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES					DOCKET NUMBER(S)																		
0	5	1	8	8	5	8	5	-	0	3	0	-	0	0	0	6	1	4	8	5						0	5	0	0	0				
0	5	1	8	8	5	8	5	-	0	3	0	-	0	0	0	6	1	4	8	5						0	5	0	0	0				

OPERATING MODE (8)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §. (Check one or more of the following) (11)									
3		20.402(b)		20.406(c)	X	60.73(a)(2)(iv)		73.71(b)			
POWER LEVEL (10)	01010	20.406(a)(1)(i)		60.36(a)(1)		60.73(a)(2)(v)		73.71(c)			
		20.406(a)(1)(ii)		60.36(a)(2)		60.73(a)(2)(vii)		OTHER (Specify in Abstract below and in Text, NRC Form 305A)			
		20.406(a)(1)(iii)		60.73(a)(2)(i)		60.73(a)(2)(viii)(A)					
		20.406(a)(1)(iv)		60.73(a)(2)(ii)		60.73(a)(2)(viii)(B)					
		20.406(a)(1)(v)		60.73(a)(2)(iii)		60.73(a)(2)(ix)					

LICENSEE CONTACT FOR THIS LER (12)	
NAME	TELEPHONE NUMBER
Merlin G. Williams - Superintendent of Regulatory, Quality and Administrative Services	AREA CODE 3 1 6 3 6 4 - 1 8 1 3 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)											
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At 2131 CDT on May 18, 1985, an Engineered Safety Features Actuation Signal (ESFAS) was initiated when a steam generator (S/G) level transmitter sensed an erroneous high-level condition. Since a redundant transmitter had previously been removed from service for a surveillance test, this resulted in a S/G two-out-of-four high level trip which caused a Feedwater Isolation Signal (FWIS) to be initiated. All required engineered safety features equipment responded properly.

The plant was in Mode 3, Hot Standby, prior to initial criticality at the time of this event. The Reactor Coolant System was at normal operating temperature and approximately 1900 psig.

The specific cause has not been determined, however investigation revealed that the most probable cause of the erroneous level indication was a pressure perturbation in a common sensing line due to a separate work activity.

This event posed no threat to the public health or safety.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
Wolf Creek Generating Station	05000482	85	030	00	2	OF 02

TEXT (If more space is required, use additional NRC Form 365A's) (17)

At 2131 CDT on May 18, 1985, an Engineered Safety Features Actuation Signal (ESFAS) was initiated when a steam generator level transmitter sensed an erroneous high-level condition.

Surveillance procedure STS IC-204, "Analog Channel Operational Test 7300 Process Instrumentation Protection Set IV (Yellow)", had previously removed AE-LT 547, a Steam Generator D (S/G "D") level transmitter, from service. As expected, this resulted in a HI-HI S/G "D" Level bistable trip for one channel (pre-trip), but not an actual ESFAS actuation since two-out-of-four channel coincidence is required to initiate a Feedwater Isolation Signal (FWIS).

Subsequent to this, a HI-HI S/G "D" Level bistable associated with level transmitter AE-LT 554 tripped. This resulted in a FWIS actuation as the necessary coincidence trip logic was satisfied. All required engineered safety features equipment responded properly.

By utilizing additional S/G "D" level indicators redundant to the two tripped channels, it was promptly determined that the event was not caused by an actual deviation in level. Transmitter AE-LT 547 was therefore placed back in service, and the actuated plant systems were restored to a normal configuration per plant operating procedures at 2137 CDT.

The plant was in Mode 3, Hot Standby, prior to initial criticality at the time of this event. The Reactor Coolant System was at normal operating temperature and approximately 1900 psig. The brief duration of this event had no appreciable affect on primary or secondary plant parameters.

The cause of the second bistable trip is not fully understood, however investigation identified a separate work activity as the most probable cause of this incident. At the time of this event, steam flow transmitter AB-FT 542 was being removed from service in support of startup test procedure SU7-0018.1, "Calibration of Steam and Feedwater Flow Instrumentation". This evolution may have caused a perturbation in the flow transmitter sensing line, and since this sensing line is common to AE-LT 554, it could have resulted in an erroneous high-level condition sufficient to cause a bistable trip. An attempt was made to duplicate this trip by repeating the steps initially taken in removing the transmitter from service, however no bistable trip occurred.

Following this event, the need to exercise extreme caution when performing transmitter valving operations was reemphasized to all I&C technicians. Particular emphasis was placed on the care required to minimize interaction between instruments which share common sensing lines. In addition, this Licensee Event Report will be assigned as required reading for all operating and I&C personnel.

There was no equipment damage or release of radioactivity as a result of this event and the event posed no threat to the public health or safety.

Previous occurrences: None



KANSAS GAS AND ELECTRIC COMPANY

GLENN L KOESTER
VICE PRESIDENT - NUCLEAR

June 14, 1985

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

COPY FOR

Mr. R.P. Denise, Director
Wolf Creek Task Force
U.S. Nuclear Regulatory Commission
Region IV
611 Ryan Plaza Drive, Suite 1000
Arlington, Texas 76011

KMLNRC 85-155

Re: Docket No. STN 50-482

Subj: Licensee Event Report 85-030-00

Gentlemen:

The enclosed Licensee Event Report is submitted pursuant to
10 CFR 50.73 (a) (2) (iv) concerning an Engineered Safety Feature
actuation.

If you have any questions concerning this matter, please contact
me or Mr. Otto Maynard of my staff.

Yours very truly,

Glenn L Koester

GLK:bb

Enc.

xc:PO'Connor (2), w/a

JCummins, w/a

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