AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-285
UNIT Fort Calhoun Station
DATE June 7, 1985
COMPLETED BY T. P. Matthews
TELEPHONE (402) 536-4733

MONTH May, 1985

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	485.1	17	484.8
2	484.7	18	483.8
3	484.1	19	482.6
4	484.0	20	481.2
5	483.9	21	480.8
6	483.4	22	480.1
7	482.9	23	479.5
8	482.4	24	479.1
9	481.0	25	478.0
10	480.8	26	476.3
11	479.6	27	475.5
12	479.2	28	476.3
13	480.0	29	476.7
14	482.3	30	475.6
15	484.3	31	475.9
16	484.5		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

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PDR ADOCK 05000285
R PDR

IE24
1/1

OPERATING DATA REPORT

DOCKET NO. 50-285
DATE June 7, 1985
COMPLETED BY T. P. Matthews
TELEPHONE (402) 536-4733

OPERATING STATUS

1. Unit Name: Fort Calhoun Station
2. Reporting Period: May, 1985
3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 502
5. Design Electrical Rating (Net MWe): 478
6. Maximum Dependable Capacity (Gross MWe): 502
7. Maximum Dependable Capacity (Net MWe): 478
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

Notes

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: None

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744.0	3,623.0	102,409.0
12. Number Of Hours Reactor Was Critical	744.0	3,596.6	78,876.8
13. Reactor Reserve Shutdown Hours	0.0	0.0	1,309.5
14. Hours Generator On-Line	744.0	3,588.7	78,256.1
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	1,111,352.9	5,299,096.5	99,485,863.5
17. Gross Electrical Energy Generated (MWH)	375,038.0	1,801,024.0	32,570,649.0
18. Net Electrical Energy Generated (MWH)	357,817.2	1,719,094.3	31,130,731.6
19. Unit Service Factor	100.0	99.1	76.4
20. Unit Availability Factor	100.0	99.1	76.4
21. Unit Capacity Factor (Using MDC Net)	100.6	99.3	66.1
22. Unit Capacity Factor (Using DER Net)	100.6	99.3	63.9
23. Unit Forced Outage Rate	0.0	0.0	3.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
1985 Refueling Shutdown is tentatively scheduled for October, 1985 with start up in December, 1985.

25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A
26. Units In Test Status (Prior to Commercial Operation): N/A

	Forecast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

(9/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH May, 1985

DOCKET NO. 50-285
UNIT NAME Fort Calhoun Station
DATE June 7, 1985
COMPLETED BY T. P. Matthews
TELEPHONE (402) 536-4733

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
									There were no unit shutdowns during the month of May, 1985.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

(9/77)

Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending May, 1985.

1. Scheduled date for next refueling shutdown. October, 1985

2. Scheduled date for restart following refueling. December, 1985

3. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? Yes

   a. If answer is yes, what, in general, will these be?

      Technical Specifications change to accommodate increased radial peaks due to further reduction in radial leakage.

   b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. _____

   c. If no such review has taken place, when is it scheduled? _____

4. Scheduled date(s) for submitting proposed licensing action and support information. September, 1985

5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

   Methodology Changes June, 1985

6. The number of fuel assemblies:

a) in the core	133	assemblies
b) in the spent fuel pool	305	"
c) spent fuel pool storage capacity	729	"
d) planned spent fuel pool storage capacity	May be increased via fuel pin consolidation	"

7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. 1996

Prepared by J K Gasper Date June 3, 1985

OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

May, 1985
Monthly Operations Report

I. OPERATIONS SUMMARY

Fort Calhoun Station operated at 100% power throughout the month of May, 1985.

Requalification and hot license simulator training at Combustion Engineering in Windsor, Connecticut, continued through May. Hot license training for two engineers and four operators continued through May.

The Maintenance and Technical departments continued preparation for the refueling outage scheduled this fall.

The Chemical and Radiation Protection group prepared high level shipments and normal shipments during May. The Auxiliary Building cleanup of stored waste will greatly improve background exposures in this area.

Security improvements continued to be implemented per schedule.

No safety valve or PORV challenges or failures occurred.

A. PERFORMANCE CHARACTERISTICS

None

B. CHANGES IN OPERATING METHODS

None

C. RESULTS OF SURVEILLANCE TESTS AND INSPECTIONS

None

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

Procedure	Description
SP-FAUD-1	Fuel Assembly Uplift Condition Detection. This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 since it only involved the evaluation of data from a surveillance test to verify that a fuel assembly uplift condition did not exist.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL (Continued)

Procedure	Description
SP-SOV-1	Periodic Cycling of Solenoid Valves Preventive Maintenance to Maintain 79-91B Qualification. This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 as it merely assured that solenoid valves which correspond to specific safety related valves have been cycled to ensure they remain qualified per the District's Electrical Equipment Qualification Program.

System Acceptance Committee Package for May, 1985:

Package	Description/Analysis
EEAR FC-84-62	Repair Damaged Stud Hole RC-2A Primary Manway. This modification restored the primary manway stud hole to its original integrity following Combustion Engineering's recommended repair procedure. This modification has no adverse effect on the safety analysis.

E. RESULTS OF LEAK RATE TESTS

During the month of May, the biannual leak rate surveillance test for the containment purge valves was completed. The total leakage for Penetrations M-87 and M-88 was 2,800 sccm. This is actually 2,322.2 sccm lower than the previous six-month test, which now decreases the Type B and C total from 11,749.73 sccm to 9,427.53 sccm.

The new B and C leak rate of 9,427.53 sccm is well below the allowed leakage of .6 La (62,951 sccm) as specified in 10CFR50 Appendix J. The next scheduled test that will affect the B and C leak rate totals is the biannual PAL door surveillance test.

F. CHANGES IN PLANT OPERATING STAFF

None

G. TRAINING

During May, three auxiliary operator (nuclear) candidates successfully completed qualification boards. Training for the six hot license candidates continued. Simulator requalification training of licensed operators continued. Annual emergency preparedness refresher training commenced.

H. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

Package	Description
Amendment No. 88	This amendment revises the Technical Specifications to add requirements related to the reactor protection system and the engineered safety features actuation system that would limit the length of time that a channel in these systems may be bypassed. If the length of time is exceeded, then the channel must be tripped.
Amendment No. 89	This amendment adds new Technical Specifications which will require OPPD to implement and maintain a program to ensure the capability to obtain and analyze a reactor coolant sample and containment atmosphere sample under accident conditions.

II. MAINTENANCE (Significant Safety Related)

None

W. Gary Gates

W. Gary Gates
Manager
Fort Calhoun Station

Omaha Public Power District
1623 Harney Omaha, Nebraska 68102
402/536-4000

June 13, 1985
LIC-85-255

Mr. James M. Taylor, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, DC 20555

Reference: Docket No. 50-285

Dear Mr. Taylor:

May Monthly Operating Report

Please find enclosed ten (10) copies of the May, 1985 Monthly Operating Report for the Fort Calhoun Station Unit No. 1.

Sincerely,

R. L. Andrews
Division Manager
Nuclear Production

RLA/TPM/dao

Enclosures

cc: NRC Regional Office
Office of Management & Program Analysis (2)
Mr. R. R. Mills - Combustion Engineering
Mr. T. F. Polk - Westinghouse
Nuclear Safety Analysis Center
INPO Records Center
American Nuclear Insurers
NRC File

IE24 1/1

Employment with Equal Opportunity
Male Female