

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Wolf Creek Generating Station DOCKET NUMBER (2) 0 5 0 0 0 4 8 2 PAGE (3) 1 OF 0 2

TITLE (4)  
ESF Actuation - Auxiliary Feedwater Actuation

EVENT DATE (5)			LER NUMBER (8)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)														
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)												
0	5	1	6	8	5	8	5	0	2	8	0	0	0	6	1	4	8	5	0	5	0	0	0

OPERATING MODE (8) 3

POWER LEVEL (10) 0 1 0 1 0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 C.F.R. § (Check one or more of the following) (11)

<input type="checkbox"/> 20.402(b)	<input checked="" type="checkbox"/> 20.406(c)	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)
<input type="checkbox"/> 20.406(a)(1)(i)	<input type="checkbox"/> 50.36(a)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(e)
<input type="checkbox"/> 20.406(a)(1)(ii)	<input type="checkbox"/> 50.36(a)(2)	<input type="checkbox"/> 50.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 350A)
<input type="checkbox"/> 20.406(a)(1)(iii)	<input type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)	
<input type="checkbox"/> 20.406(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)	
<input type="checkbox"/> 20.406(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME Merlin G. Williams - Superintendent of Regulatory, Quality and Administrative Services TELEPHONE NUMBER 3116 3164-1818311

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRRDS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)  NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At 1515 CDT on May 16, 1985, an Auxiliary Feedwater Actuation Signal (AFAS) was initiated when level in Steam Generator "D" (S/G "D") decreased below the low-low level actuation setpoint due to the combined effects of bleeding steam via the S/G "D" Atmospheric Relief Valve, and the "shrinking" of level caused by an increase in the rate of auxiliary feedwater introduction. All required engineered safety features equipment responded properly.

The plant was in Mode 3, Hot Standby, prior to initial criticality at the time of this event. The Reactor Coolant System was at normal operating pressure and temperature.

The Auxiliary Feedwater System was restored to its pre-actuation line-up per plant procedures at 1521 CDT.

There was no damage to plant equipment or release of radioactivity as a result of this event. At no time did this event pose a threat to the public health or safety.

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)  Wolf Creek Generating Station	DOCKET NUMBER (2)  0 5 0 0 0 4 8 2	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 5	- 0 2 8	- 0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

At 1515 CDT on May 16, 1985, an Auxiliary Feedwater Actuation Signal (AFAS) was initiated when level in Steam Generator "D" (S/G "D") decreased below the low-low level actuation setpoint.

The plant was in Mode 3, Hot Standby, prior to initial criticality at the time of this event. The Reactor Coolant System (RCS) was at normal operating pressure and temperature, with four Reactor Coolant Pumps in operation. Feedwater was being supplied to S/G "D" via Auxiliary Feedwater Pump "B". RCS average temperature (Tavg) was being reduced via the bleeding of steam from the S/G "D" Atmospheric Relief Valve.

As the water level in S/G "D" decreased toward the low-low level AFAS setpoint, plant operating personnel increased the rate of auxiliary feedwater introduction to S/G "D". The immediate affect of this increased flow of cold water was to further "shrink" S/G "D" level, which caused the low-low level setpoint to be exceeded, and resulted in an AFAS actuation. All required engineered safety features equipment responded properly.

Following actuation, Auxiliary Feedwater Pump "A" increased water levels in S/G's "B" and "C" by approximately 5 percent to a level of 55 percent. Level in S/G "D" was manually increased to approximately 57 percent following the event via Auxiliary Feedwater Pump "B" which was already in operation. These S/G level increases caused a corresponding decrease of approximately 3 degrees F in Tavg to 554 degrees F. No appreciable change in S/G "A" level occurred during the event as Auxiliary Feedwater Pump "B" flow to S/G "A" was isolated since steam was initially being drawn from S/G "D" only.

The Auxiliary Feedwater System was returned to the pre-actuation lineup per plant procedure at 1521 CDT.

The cause of the event was a cognitive personnel error, in that the rate of increase in auxiliary feedwater flow was excessive considering the close proximity of steam generator level to the AFAS setpoint. This error has been discussed with the operating personnel in shift briefings and crew meetings stressing operator awareness of plant status and particularly early recognition of developing trends. In addition, this Licensee Event Report will be assigned as required reading for all operating personnel.

There was no damage to plant equipment or release of radioactivity as a result of this event. At no time did this event pose a threat to the public health or safety.

Previous occurrences: None



KANSAS GAS AND ELECTRIC COMPANY

GLENN L. KOESTER  
VICE PRESIDENT - NUCLEAR

June 14, 1985

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555

COPY FOR

Mr. R.P. Denese, Director  
Wolf Creek Task Force  
U.S. Nuclear Regulatory Commission  
Region IV  
611 Ryan Plaza Drive, Suite 1000  
Arlington, Texas 76011

KMLNRC 85-154

Re: Docket No. STN 50-482

Subj: Licensee Event Report 85-028-00

Gentlemen:

The enclosed Licensee Event Report is submitted pursuant to 10 CFR 50.73(a) (2) (iv) concerning an Engineered Safety Feature actuation.

If you have any questions concerning this matter, please contact me or Mr. Otto Maynard of my staff.

Yours very truly,

*Glenn L. Koester*

GLK:bb

Enc.

xc:PO'Connor (2), w/a

JCuramins, w/a

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