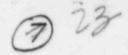


NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555



MAR 1 2 1984

MEMORANDUM FOR:

R. Wayne Houston, Assistant Director

for Reactor Safety

Division of Systems Integration

FROM:

Brian W. Sheron, Chief Reactor Systems Branch

Division of Systems Integration

SUBJECT:

GRAND GULF UNIT 1 TECHNICAL SPECIFICATIONS

Reactor Systems Branch has reviewed the Grand Gulf Technical Specifications revised through Amendment 12. Enclosure 1 lists the sections which we reviewed. Enclosure 2 provides our comments.

Brian W. Sheron, Chief Reactor Systems Branch

Division of Systems Integration

Enclosures: As stated

cc: D. Hoffman

RSB Section B Members

R. Mattson R. Capra

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ENCLOSURE 1

TECHNICAL SPECIFICATIONS, BASES AND DEFINITIONS REVIEWED BY THE REACTOR SYSTEMS BRANCH FOR GRAND GULF UNIT 1

I.	Definition	
	1.3 1.8 1.12 1.13 1.21 1.24	Average Planar Linear Heat Generation Rate Critical Power Ratio Emergency Core Cooling System (ECCS) Response Time End-of-Cycle Recirculation Pump Trip System Response Time Linear Heat Generation Rate Minimum Critical Power Ratio
II.	Safety Limits and Limiting Safety System Settings	
	2.1.1 2.1.2 2.1.3 2.1.4 2.2.1	Thermal Power, Low Pressure or Low Flow Thermal Power, High Pressure and High Flow Reactor Coolant System Pressure Reactor Vessel Water Level Reactor Protection System Instrumentation Setpoints
II.	Limiting C	onditions for Operation and Surveillance Requriements
	3/4.2.3 3/4.3.1 3/4.3.2 3/4.3.3 3/4.3.4 3/4.3.5 3/4.3.7.5 3/4.3.7.5 3/4.3.8 3/4.4.1 3/4.4.2 3/4.4.6.2 3/4.4.7 3/4.4.9 3/4.7.3 3/4.9.8 3/4.9.11 3/4.10.4	Reactor Protection System Instrumentation Isolation Actuation Instrumentation Emergency Core Cooling System Actuation Instrumentation Recirculation Pump Trip Actuation Instrumentation/ATWS Recirculation Pump Trip System Instrumentation Reactor Core Isolation Cooling System Actuation Instrumentation Remote Shutdown Monitoring Instrumentation Accident Monitoring Instrumentation Plant Systems Actuation Instrumentation Recirculation System Safety Valves Reactor Steam Dome Main Steam Isolation Valves Residual Heat Removal Emergency Core Cooling System Refueling Operations; Water Level - Reactor Vessel Refueling Operation; Residual Heat Removal and Coolant Circulation Special Test Exceptions; Recirculation Loops
īv.	Bases	Renetal Coaput Stron

Bases for all the areas identified in Sections II and III above.

COMMENTS ON GRAND GULF UNIT 1 TECHNICAL SPECIFICATIONS

1) 2.2.1 Reactor Protection System Instrumentation Setpoints

Table 2.2.1-1 Item 3, "Reactor Vessel Steam Dome Pressure-High;" The allowable value should be less than or equal to 1045 psig (Reference page 5-5 of the SER).

2) Bases 2.1.2 Thermal Power, High Pressure and High Flow

The last paragraph of the text lists a GE report number as NEDO-203040. The correct report number is NEDO-20340.

3) 3.4.2.2 APRM Setpoints

The time constant for the thermal power monitor needs to be included in the LCO's and surveillance requirements. (Reference page 15-4 of Grand Gulf SER).

- 4) 3/4.3.3 Emergency Core Cooling System Actuation Instrumentation
 - Table 3.3.3-2 item B.2.e "LPCI Pump B and C Discharge Pressure -High" should have the same allowable value as item A.2.f "LPCI Pump A Discharge Pressure - High" which is 115-135 psig, increasing.
 - 2. Table 3.3.3-3 "Emergency Core Cooling System Response Times" gives the LPCI A & B Pump response time as < 45 seconds. Table 6.3-1 of the FSAR states that LPCI rated flow is achieved in < 40 seconds for the DBA analysis. The specification should be changed to read < 40 seconds.</p>