



Carolina Power & Light Company

Brunswick Nuclear Plant
P. O. Box 10429
Southport, N.C. 28461-0429

JAN 05 1993

FILE: B09-13510C
SERIAL: BSEP-93-0002

10CFR50.73

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D. C. 20555

BRUNSWICK STEAM ELECTRIC PLANT UNIT 2
DOCKET NO. 50-324
LICENSE NO. DRP-62
LICENSEE EVENT REPORT 2-92-010

Gentlemen:

In accordance with Title 10 of the Code of Federal Regulations, the enclosed Licensee Event Report is submitted. This report fulfills the requirement for a written report within thirty (30) days of a reportable occurrence and is submitted in accordance with the format set forth in NUREG-1022, September 1983.

Very truly yours,

J. M. Brown, Plant Manager - Unit 2
Brunswick Nuclear Plant

TMJ/

Enclosure

cc: Mr. S. D. Ebnetter
Mr. R. H. Lo
BSEP NRC Resident Office

110023

9301110213 930105
PDR ADOCK 05000324
S PDR

EXPIRES: 6/31/95

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 60.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNB 7314), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20556-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)

Brunswick Steam Electric Plant, Unit 2

DOCKET NUMBER (2)

05000324

PAGE (3)

1

TITLE (4)

PENETRATION LEAKAGE IN EXCESS OF TECHNICAL SPECIFICATION ALLOWABLE LIMIT DURING LOCAL LEAK RATE TESTING

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
12	07	92	92	- 10 -	00	01	05	93	FACILITY NAME	DOCKET NUMBER
										05000
									FACILITY NAME	DOCKET NUMBER
										05000

OPERATING MODE (9)	4	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following)(11)								
POWER LEVEL (10)	000	20.402(b)		20.405(c)		50.73(a)(2)(iv)		73.71(b)		
		20.405(a)(1)(i)		50.36(c)(1)		X	50.73(a)(2)(v)		73.71(c)	
		20.405(a)(1)(ii)		50.36(c)(2)			50.73(a)(2)(vi)		OTHER	
		20.405(a)(1)(iii)		50.73(a)(2)(i)			50.73(a)(2)(viii)(A)		(Specify in Abstract and Text)	
		20.405(a)(1)(iv)	X	50.73(a)(2)(ii)			50.73(a)(2)(viii)(B)			
		20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(ix)				

LICENSEE CONTACT FOR THIS LER (12)

NAME

Theresa M. Jones, Regulatory Compliance Specialist

TELEPHONE NUMBER

(919) 457-2039

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRCDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRCDS
X	SE	ISV	R344	Y					

SUPPLEMENTAL REPORT EXPECTED (14)

X YES (If yes, complete EXPECTED SUBMISSION DATE)		NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
				03	19	93

ABSTRACT (Limit to 1400 spaces, i.e. approximately fifteen single space typewritten lines) (16)

On December 07, 1992, the Unit 2 reactor was shutdown in day 230 of a maintenance outage. Type C local leak rate testing (LLRT) of the main steam isolation valves (MSIVs) had been performed. The results of testing indicated that MSIV leakage on main steam line (MSL) D exceeded the Technical Specification (TS) limit of 11.5 scfh. MSLs A, B, and C tested satisfactorily. Trouble shooting and analysis of the failure mechanism are in progress. To date work has been focused on the outboard MSIV. Preliminary findings indicate that the valve appears to have had excessive disk piston to valve bore clearance and an alignment problem (i.e., the main valve disc is not guiding properly into the in-body seat). Subsequent to repairs, the outboard MSIV will be tested a minimum of two times to establish repeatability of the results.

The loss of the primary containment isolation function of D MSL is a potentially safety significant event.

LER 2-91-019 reported the failure of both MSL C and D on November 12, 1991. The associated MSIVs were repaired and the unit was returned to service in January of 1992 until April 21, 1992 when it was shutdown for the current outage.