

GULF STATES UTILITIES COMP.

HIVER BEND FTATION HOST OFFICE POR 225 BT FRANCISSFLLE LOUISIANA 70*95 AREA CODE 504 835-5056 3446-5651

> January 4, 1913 RBG- 37966 File Nos. G9.5, G9.25.1.3

U.S. Nuclear Regulatory Commission Document Control Desk Washington, D.C. 20555

Gentlemen:

River Bend Station - Unit 1 Docket No. 50-458

Please find enclosed Licensee Event Report No. 92-028 for River Bend Station -Unit 1. This report is submitted pursuant to 10CL-R50.73.

Sincerely,

W. H. Odell Manager - Oversight River Bend Nuclear Group

LE/JPS/FRC/DCH/VCC/kvm

080075

9301080268 9 ADOCK 05000458 PDR

C: U.S. Nuclear Regulatory Commission
611 Ryan Plaza Drive, Suite 400
Arlington, TX 76011

NRC Resident Inspector P.O. Box 1051 St. Francisville, LA 70775

INPO Records Center 1100 Circle 75 Parkway Atlanta, GA 20339-3064

Mr. C.R. Oberg Public Utility Commission of Texas 7800 Shoal Creek Blvd., Suite 400 North Austin, TX 78757

Department of Environmental Quality Radiation Protection Division P.O. Box 82135 Baton Rouge, LA 70884-2135 ATTM: Administrator

(See 1 FACILITY NAME (.) TITLE (0) NUCT		ENSEI	E EVENT RI	EPORT (LD			ENTRATED B	Innini in		this present		
FACILITY NAME (1)		w. nestern	ed number of digi	ts/oberapters	for each bi			INFORMATION COMMENTS RE AND RECORDS REGULATORY C THE PAPERWO NANAGEMENT	COLLECTION GARDING BL MANAGEME 7 MM SEIDN RK. REDUCT NND BL/DOFT	HEARSON SET IPECILEST IPECILEST IPECINE IPECIECT WATCHINGTON WATCHINGTON	50.0 HRS 50.0 HRS 7E TO TH NB8 7714 00 2055 (2150-010 L DC 2055)	LY WITH TH E PORWAR E INFORMATIO U.S. NUICLES 5-0001, AN(1-1 40, OFFICE (1	
TITLE (4) NUCL	RIV	ER B	END STATI	ON				DOCKET NUME	EH (2) 05000	458		PAGE (3) 1 GF 4	
SUPERVISO	EAR DR TO	INST REC	BUMFET ATA	ON INOL AT SUR	ZERABL ZEILLA	E DU NCE	E TO REQUI	FAILURE	OF A WERE	SHIFI NOT M	nernes Jenne V IET	dhareenna mina	
EVENT DATE			LEA NUMBER	(6)	REPOR	TNUM	3ER (7)	1001	HER FAC	LITIES INV	OLVED (8)	
MONTH DAY			nikovencha Novench	ARS/IDION NOMBRE	MONDA	UAY	YEAR	FACELITY NAME		05000			
12 07	92	92	- 028	- 00	01	04	93	FACILITY NAME		DOCKET NO 05000			
MODE (9)	3	THIS RI	EPORT IS SUBMI	TTED PURS	120 405%	RE REQ	UIREME	INTS OF 10 C	FR 4: (Ch	eck one or	mare) (1	1) 71(b)	
POWER	or manager of	20.4	(05(a)(1)(l)		50.58(0)	(1)		50.7	50.73(e)(2)(v)		73	71(c)	
LEVEL (10)	U .	20.4			50.36(0)	2)		50.7	50.73(a)(2)(vii)		1 101	OTHER	
		20.405(a)(1)(bi) 20.405(a)(3)(b)			X 50.73(n)(2)(i) 50.73(n)(ii)(ii)			50.73(a)(2)(viii)(A)			(lipsonty = abstract		
								50.73(a)(2)(vili)(B))(6)	Form Stilles		
		20.4			50.73(a)			50.7	50.73(a)(2)(x)				
				LICENSEE	CONTACT	FOR TH	IS LER	(12)			t fan strivere	national and a state of the	
unne Les /	A. El	(GLAN	D, DIRECT	OR - NI	JCLEAR	LIC	ENSIN	iG (504)	381-41	45		
	-	COMPL	ETE ONE LINE FO	OR EACH CO	MPONENT	FAILUF	F. DESC	CAIBED IN TH	IS REPOR	T (13)			
CAUTER SYSTEM	срие	NENT	MANT OF MELTINER	REPORTABLE TO NRIVER			36UAC	SPESTION CL	MFONENT	MANUFACT	IREA	REPORTAIN.E	
										1.6.2			
		aintendiren de				T						an an a' de anna an a	
ser conducer reaches		nini nampa	IDAL PAPALANAL D				ning and a second	hand the second s	maganteens	lamonanicas	groanicases	garmenigues o	
VEN IN yes, cheropiele k	VPECTED	6.08Million	N DATES	EF JAT EXP	K NO		****		EX SUE	PECTED	MONTH	DWA. AFW	
ABSTRACT (Lim)	1 to 140	o spaces	i e., approximate	ly 15 single-r	l paced type	written li	nes) (16	1999 - 1999 - 1999 - 1999 - 1999 - 1999 - 1999 - 1999 - 1999 - 1999 - 1999 - 1999 - 1999 - 1999 - 1999 - 1999 - 5)		ATE (15)	ana na seriena	harvenderen	

kept current and the failure to take actions required by Technical Specifications 3.3.1 and 3.3.7.6. Therefore, this report is submitted pursuant to 10CFR50.73(a)(2)(i)(B) as operation prohibited by the plant Technical Specifications.

The root cause of this event was failure of the Shift Supervisor to recognize that the surveillance requirements were not current. Corrective octions include modifying the applicable procedures to alert operators to SRM/IRM operability, training, and an evaluation by Plant Management of the need for disciplinary action.

During this event, all rods remained fully inserted in the core and the reactor mode switch remained in the shutdown position. When the STPs were performed, all data was acceptable with no adjustments required.

REQUIRED NUMBER OF DIGITS/CHARACTERS FOR EACH BLOCK

BLOCK NUMBER	NUMBER OF DIGITS/CHARACTERS	TITLE				
	UP TO 46	FACILITY NAME				
2	8 TOTAL 3 IN ADDITION TO 05000	DOCKET NUMBER				
3	VARIES	PAGE NUMBER				
4	UP TO 76	TITLE				
5	6 TOTAL 2 PER BLOCK	EVENT DATE				
6	7 TOTAL 2 FOR YEAR 3 FOR SEQUENTIAL NUMBER 2 FOR REVISION NUMBER	LER NUMBER				
7	6 TOTAL 2 PER BLOCK	REPORT DATE				
B	UP TO 18 FACILITY NAME 8 TOTAL DOCKET NUMBER 3 IN ADDITION TO 05965	OTHER FACILITIES INVOLVED				
9	t	OPERATING MODE				
10	3	POWER LEVEL				
11	1 CHECK BOX THAT APPLIES	REQUIREMENTS OF 10 CFR				
12	UP TO 50 FOR NAME 14 FOR TELEPHONE	LICENSEE CONTACT				
13	CAUSE VARIES 2 FOR SYSTEM 4 FOR COMPONENT 4 FOR MANUFACTURER NPRDS VARIES	EACH COMPONENT FAILURE				
- 14	CHECK BOX THAT APPLIES	SUPPLEMENTAL REPORT EXPECTED				
15	6 TOTAL 2 PER BLOCK	EXPECTED SUBMISSION DATE				

NRC FORM 366A U.S. NUCLI	APPROVED BY OMB NO. 3150-0104 EXPIRES 5/31/95 ESTIMATED RURDEN PER RESPONSE 10 20MPLY WITH THIS INFORMATION COLLECTION REQUEST: BUD HRS FORWARE COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAN REQULATORY COMMISSION, WARHINGT, JN, DC 20558 0001 AND TO THE PAPERWORK REDUCTION PROJECT 3150-0104, OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503					
LICENSEE EVENT REPO TEXT CONTINUATIO						
AND A REAL PROPERTY A REAL PROPERTY AND A REAL	DOCKET NUMBER (2)	LER NUMBER (8)			PAGE (3)	
and a second	aliyaani aa ahaa ahaa ahaa ahaa ahaa ahaa ah	YEAR	SEQUENTIAL NUMBER	REVISION		
RIVER BEND STATION	05000 458	92	92 " 028 " 00		2.0*4	

REPORTED CONDITION

On 12/07/92 at 0630 with the plant shutdown for a planned outage (Operational Condition 3 - Hot Shutdown), it was discovered that all source range and intermediate range monitors (SRMs and IRMs) of the nuclear instrumentation system were inoperable. This condition was due to weekly surveillance requirements that were not kept current and the failure to take actions required by Technical Specifications 3.3.1 action 'b' and 3.3.7.6 action 'b'. Therefore, this report is submitted pursuant to 10CFR50.73(a)(2)(i)(B) as operation prohibited by the plant Technical Specifications.

INVESTIGATION

On 12/06/92, a shutdown was in progress in preparation to repair the 'B' reactor recirculation pump seal. At 1729 a manual reactor scram was inserted and the reactor mode switch was placed in the shutdown position. Abnormal operating procedure (AOP)-0001 ("REACTOR SCRAM") was entered and all immediate operator actions were completed including a verification that all rods were fully inserted. Completion of the immediate actions of AOP-0001 was documented in the control room log book on 12/06/92 at 1731.

When the reactor mode switch was placed in the shutdown position, this placed the plant in Operational Condition 3, Hot Shutdown. Per Technical Specification 3.3.1 "Reactor Protection System Instrumentation", the intermediate range monitors (IRMs) are required to be operable in Operational Condition 3 with a minimum of 3 instruments per trip system. At this time the weekly functional surveillance test procedure (STPs) for the IRMs were not current. With these STPs not current, all IRMs were inoperable. The fact that the IRMs were inoperable was not noted by Operations personnel until 0630 on 12/07/92. Technical Specification 3.3.1 action 'b' requires that with less than the minimum number of channels operable in both trip systems, that one trip system be placed in the tripped condition within one hour and that the required actions per Table 3.3.1-1 be taken.

Similarly, the source range monitors (SRMs) are required to be operable in Operational Condition 3 per Technical Specification 3.3.7.6. As with the IRMs, the weekly functional STPs were not current musing the SRMs to be inoperable. Operations personnel noted this also at 0630 on 12/07/52, the same time it had been discovered that the TRMs were inoperable.

The required actions of both Technical Specifications 3.3.1, action 'b' and 3.3.7.6, action 'b' are to verify within 1 hour that all insertable control rods are inserted into the core, and to lock the reactor mode switch in the shutdown position (by removing the key). These requirements were satisfied at

NRC FORM 366A	U.S. NUCLEAR REGULATORY COMMISSION				APPROVED BY OMB NO. 3150-0104 EXPIRES 5/31/95				
LICENSEE E	EETIMATED BURDEN PER RESPONSE TO DOMPLY WITH THINFORMATION COLLECTION REQUEST 50.0 HTML FORWATION COLLECTION REQUEST 50.0 HTML FORWATIC AND RECORDS MANAGEMENT BRANCH (MMBB 7714, U.S. NUCLEY REGUL, CHY COMMESION, WASHINGTON, F.J. 20555-0001 AND THE PAPERWORK REDUCTION PROJECT (\$150-0104, OFFICE (MANAGEMENT AND BUDGET WASHINGTON, DC 20053								
FACILITY NAME	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)				
			YEAR	SECUENTIAL NUMBER	REVISION NUMBER				
RIVER BEND STATIO	END STATION		92	- 028 -	00	3 0 4			

0636 on 12/07/92 as documented by limiting condition for operation (LCO) 92-629. During the period from 1829 on 12/06/92 until 0636 on 12/07/92 the requirements of Specification 3.3.1 action 'b' were not met; however, all rods remained inserted into the core and the reactor mode switch remained in the shutdown position. At 1400 on 12/07/92 all required IRM STPs were completed and action 'b' of Specification 3.3.1 was exited. The weekly functional STPs for the SRMs were started at 1352 on 12/07/92. At 1434 the minimum number of SRMs were operable and at 2112, on 12/07/92, all SRMs were operable. With the required SRMs operable, action 'b' of Specification 3.3.7.6 was exited.

Technical Specification 3.3.1 action 'b' and 3.3.7.6 action 'b' were not entered at the proper time due to a failure of the Shift Supervisor to recognize that the IRM and SRM STPs were not current. The Shift Supervisor reviewed the outage schedule and found that the nuclear instrument STPs were not scheduled to begin until the day shift on 12/07/92. From this information, he assumed that the STPs were still current. The "Surveillance Test Events - Shutdown/Startup Logic" sheet listed the last time these STPs were performed and had it been reviewed by the Shift Supervisor this error could have been prevented.

ROOT CAUSE

The root cause in this incident was failure of the Shift Supervisor to recognize that the SRM and IRM STPs were not current at the time of entry into Operational Condition 3 (Hot Shutdown). This error was caused by reviewing only the Outage Schedule instead of using the "Surveillance Test Events - Shutdown/Startup Logic" sheet which would have shown these STPs to be past their due date.

A recent event reported in LER 92-027, described a violation of TS 3.0.4 involving an incorrect interpretation of TS 3.7.3. TS 3.7.3 requires that the reactor core isolation cooling system (RCIC) be operable prior to increasing reactor vessel pressure above 150 psig. LER 92-027 has similarities to the event reported in this LER (LER 92-028). In LER 92-027, the Shift Supervisor (SS) and Administrative Control Operating Foreman (Admin COF) failed to recognize that raising reactor vessel pressure above 150 psig placed the plant in violation of the specified applicability conditions of TS 3.7.3. Similarly, in LER 92-028, the SS did not realize that the STPs were not current. While these events have similarities, there are differences in development of the events and in the contributing factors. In LER-027, there was some confusion over a footnote in TS 3.7.3 and a Control Operating Foreman failed to challenge the SS on his decision to not place the RCIC system in the standby lineup as required by procedure. In LER 92-028, the major contributor to the event was the reliance on the outage schedule by the SS. He failed to review the proper documentation to

NRC FORM 366A

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB NO. 3150-0104 EXPIRES 5/31/95

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEIT SO & HRS. FORWARE COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MMBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (\$150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION			
RIVER BEND STATION	05000 458	92 -	028 -	00	4 4		

TEXT (II more space is required, use additional copies of NRC Form 365A) (17)

assure that the STPs were current. Note that different Shift Supervisors were involved in these two cases. Thus, different circumstances and contributors combined with improper decisions resulted in Technical Specification violations in each case.

CORRECTIVE ACTION

The following corrective actions will be taken:

- As previously described in LER 92-027, GSU is developing a case study concerning the event described in that report. The case study will be analyzed by each operating crew as an integral part of training to specifically address Technical Specification violations, including this event (LER 92-028).
- 2. The need for disciplinary action will be evaluated and administered by Plant Management.
- 3. As an enhancement to prevent recurrence, change notices will be initiated to general operating procedure (GOP)-0002, "POWER DECREASE/PLANT SHUTDOWN" and GOP-0003, "SCRAM RECOVERY," to include a caution to remind personnel that if the plant has been operating for greater than 7 days that the SRM/IRM STPs are not current and to add a step for SS/COF to verify that the SRMs and IRMs are operable.

SAFETY ASSESSMENT

3

During the period of time that the SRM/IRMs were not operable, all SRMs and IRMs were fully inserted into the core, as required by subsequent operator action 5.2 of abnormal operating procedure (AOP)-0001, "REACTOR SCRAM" and therefore, were monitoring reactor power level. All rods remained fully inserted in the core and the reactor mode switch remained in the shutdown position. When the STPs were performed on the SRMs and IRMs, all data was acceptable with no adjustments required.