

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Browns Ferry - Unit 3										DOCKET NUMBER (2) 0 5 0 0 0 2 9 6 1 OF 0 2										PAGE (3) 1 OF 0 2					
TITLE (4) Containment Isolation Initiation																									
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)															
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)												
0	4	2	9	8	5	8	5	0	1	4	0	0	0	5	2	4	8	5	0	5	0	0	0		
OPERATING MODE (9) N		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)																							
POWER LEVEL (10) 0 0 0		20.402(b)				20.405(e)				<input checked="" type="checkbox"/> 50.73(a)(2)(iv)				73.71(b)											
		20.405(a)(1)(i)				50.38(c)(1)				50.73(a)(2)(v)				73.71(c)											
		20.405(a)(1)(ii)				50.38(c)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)											
		20.405(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)															
		20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)															
		20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(ix)															
LICENSEE CONTACT FOR THIS LER (12)																									
NAME R. C. Steele														TELEPHONE NUMBER 2 0 5 7 2 9 1 - 3 5 8 3											
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																									
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS															
SUPPLEMENTAL REPORT EXPECTED (14)														EXPECTED SUBMISSION DATE (15)				MONTH	DAY	YEAR					
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)														<input checked="" type="checkbox"/> NO											

ABSTRACT (Limit to 1400 spaces; i.e., approximately fifteen single-space typewritten lines) (16)

During performance of instrument check surveillance instruction, RI-90-142 was taken to the "zero" position to check recorder reading resulting in an inadvertent initiation of group 6 containment isolation logic. A caution note was added to the applicable plant surveillance instruction to prevent similar occurrences in the future. The containment isolation signal was reset and equipment returned to normal after verifying the affected safety systems availability.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
Browns Ferry - Unit 3	0500029685	-	014	-	0002	OF 02

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Unit 1 was in cold shutdown, unit 2 was in refueling outage, and unit 3 was in cold shutdown.

On April 29, 1985, during performance of an instrument check surveillance instruction on unit 3, radiation indicator (RI-90-142) was taken to the "zero" position to verify the reading. For this particular instrument, this action will cause a trip of group 6 initiation logic. When the mode switch was placed in the "zero" position an upscale trip was accordingly generated. The surveillance instruction did not caution the operator about the possibility of generating a containment isolation signal.

The upscale trip is designed to initiate isolation of the reactor building ventilation (VA), isolation of drywell control air system (VB), isolation of the control room ventilation (VI), and starting of the standby gas treatment system (BH). The equipment functioned as designed. Initiation of containment isolation was inadvertent in that there were no precautionary notes in the instrumentation check surveillance instruction. The containment isolation signal was reset and equipment returned to normal after verifying the affected safety system availability.

To prevent similar events from occurring in the future, the instrumentation check surveillance instruction was revised to include a caution statement and a reference to reactor building ventilation radiation monitors surveillance instruction.

Responsible Plant Section - N/A

Previous Events - None

TENNESSEE VALLEY AUTHORITY

Browns Ferry Nuclear Plant

P. O. Box 2000

Decatur, Alabama 35602

May 24, 1985

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Dear Sir:

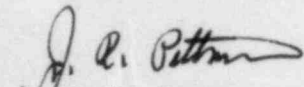
TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT (BFN) UNIT 1 -
DOCKET NO. 50-~~259~~²³⁶ - FACILITY OPERATING LICENSE DPR-33 - REPORTABLE
OCCURRENCE REPORT BFRO-50-~~259~~²³⁶/85014

24/85014

The enclosed report provides details concerning containment isolation
initiation. This report is submitted in accordance with 10 CFR 50.73
(a)(2)(iv).

Very truly yours,

TENNESSEE VALLEY AUTHORITY


G. T. Jones

Plant Manager

Browns Ferry Nuclear Plant

Enclosures

cc (Enclosures):

Regional Administrator
U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region II
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Atlanta, Georgia 30303

INPO Records Center
Suite 1500
1100 Circle 75 Parkway
Atlanta, Georgia 30339

NRC Resident Inspector, BFN