LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

MAY 1985

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373 LICENSE NO. NPF-11

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8506180325 850531 PDR ADOCK 05000373 R PDR

I. INTRODUCTION

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- II. MONTHLY REPORT FOR UNIT ONE
 - A. Summary of Operating Experience
 - B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY RELATED MAINTENANCE
 - Amendments to Facility License or Technical Specifications
 - Facility or Procedure Changes Requiring NRC Approval
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 - 2. BCCS System Outages
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 - Major Changes to Radioactive Waste Treatment System

INTRODUCTION

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Comapny and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to an artificial cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy, and the primary construction contractor was Commonwealth Edision Company.

Unit one was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982, and commercial power operation was commenced on January 1, 1984.

This report was compiled by Richard J. Rohrer, telephone number (815)357-6761 extension 575.

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11. MONTHLY REPORT FOR UNIT ONE

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT ONE

May 1-31

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May	1, 0000 hours.	Reactor critical at 98% power, generator on-line.
May	1, 0700 hours.	Reactor power reduced to 78% for LTS-1100-4, Scram Timing
		for CRD 22-59.
May	2, 0700 hours.	Reactor power restored to 96%.
May	6, 1645 hours.	Reactor power reduced to 84%, to manipulate control rods.
May	7, 0700 hours.	Reactor power restored to 96%,
May	10, 2300 hours.	Reactor power at 95%.
May	11, 0200 hours.	Reactor power reduced to 76% to manipulate control rods.
May	11, 2300 hours.	Reactor power restored to 94%.
May	19. 0200 hours.	Reactor power reduced to 58% to allow maintenance on
		heater drains.
May	20, 1500 hours.	Reactor power restored to 89%.
May	20, 1942 hours.	Reactor power reduced to 86% due to Number 3 Control Valve
		oscillations.
May	24, 0700 hours.	Reactor power at 84%.
May	24, 0855 hours.	Reactor power reduced to 74% due to trip of "B" circulating
		Water Pump.
May	24, 1500 hours.	Reactor power at 84%.
May	30, 2352 hours.	Reactor power reduced to 78% for surveillance LOS-AA-W1.
May	31, 0700 hours.	Reactor power at 84%.
May	31, 1945 hours.	Reactor manually scrammed following flooding of Lake Screen
		House when "B" Circulating Water Pump Discharge Line leaked.

The reactor was critical for 739.8 hours during May.

PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

- Amendments to facility license or Technical Specification. Three ammendments were made to the Unit one operating license and Technical Speciafications during May.
 Ammedment 21 modifies the calculational technique used in determining duct heater performance for the standby Gas Treatment System. Ammendment 22 changes the channel check requirements to accommodate new instruments installed by Environmental Qualification modifications. Ammendment 23 incorporates the requirements of 10 CFR 50.73.
 - Pacility or procedure changes requiring NRC approval.
 There were no facility or procedure changes requiring NRC approval during this reporting period.
 - Tests and Experiments requiring NRC approval.
 There were no tests or experiments requiring NRC approval during this reporting period.
 - 4. Corrective maintenance of safety related equipment. The following table (Table 1) presents a summary of safety-related maintenance completed on Unit One during the reporting period. The headings indicated in this summary include: Work Request number, Component Name, Cause of Malfunction, Results and Effects on Safe Operation, and Corrective Action.

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CORRECTIVE MAINTENANCE OF SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS	CORRECTIVE ACTION ON SAFE OPERATION
L41793	Division I Switch- gear Heat Removal Outside Air Damper.	Faulty Actuator	Damper would not stroke	Sent actuator to vendor for repair. Reinstalled and verified operability
L45261	Control Room Zone Mixing Damper "A".	Actuator Shaft is not coupled to damper.	Damper would not operate.	Connected shaft to damper.
L48324	Ammonia detector "B" for "B" Control Room Ventilation.	Out-of-calibration.	Detector alarmed and actuated ESF's.	Recalibrated detector.
L48347	Sequence of Events Recorder.	Swapped Alarm messages for Chlorine and Ammonia Alarms	Wrong Alarm messages printed.	Corrected Alarm messages.
L48478	Unit One Stand-by Gas Treatment System Recorder.	Flow Recorder out-of- calibration.	Wrong flow indication.	Recalibrated Recorder and adjusted recorder scale.
L48535	Scram pilot valve for CRD 22-59.	Valve hung up in an inter- mediate position.	Inadvertantly Scrammed this rod during half-scram testing.	Replaced scram pilot valves.
L48580	1A RHR Service Water Pump.	Faulty Motor.	Pump would not work as designed.	Replaced motor with motor from 2B RHR service water pump.
L48651	Ammonia detector "B" for "A" Control Room Ventilation.	Detector out-of-calibration	Detector alarmed and actuated ESF's.	Recalibrated detector.
L48846	"A" Rod Block Monitor	Blown fuse.	Generated Spurious Rod Blocks.	Replaced fuse.

CORRECTIVE MAINTENANCE OF SAFETY RELATED EQUIP.MENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS	CORRECTIVE ACTION ON SAFE OPERATION
L48918	"A" Standby Liquid Control Relief Valve.	Valve Leaked through port hole.	Injection of Liquid Control solution would have been slightly slower an otherwise, if required.	Cleaned and adjusted valve. Installed new bellows and lapped the seat.
L49054	HCU Accumulator for CRD 22-23.	Leaky fill valve.	Potential to cause failure to scram this rod if combined with other events.	Replaced valve.
L49059	"A" Rod Block Monitor.	Faulty power supply.	Failed downscale.	Replaced power supply and fuses tested.

C LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit One, occurring during the reporting period, May 1 through May 31, 1985. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

Licensee Event Report Number	Date	Title of Occurrence	
85-037-00	4-17-85	Division II ADS Inop.	
85-038-00	4-12-85	Ammonia Detector Actuation.	
85-039-00	4-19-85	Ammonia Detector Actuation.	
85-040-00	4-23-85	Ammonia Detector Failure.	
85-041-00	5-5-85	Spurious Auto-start of Emergency Make-up Fan.	
85-042-00	5-5-85	Ammonia/Chlorine Detector Actuations.	
85-043-00	5-8 85	Chlorine Detector Actuation.	

D. DATA TABULATIONS

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The following data tabulations are presented in this report:

- 1. Operating Data Report
- 2. Average Daily Unit Power Level
- 3. Unit Shutdowns and Power Reductions

DOCKET NO. 050-373 UNIT LaSalle One DATE June 10, 1985 COMPLETED BY Richard J. Rohrer TELEPHONE (815)357-6761

OPERATING STATUS

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1.	REPORTING PERIOD: MAY, 1985 GROS	S HOURS IN	REPORTING PE	RIOD: 744
2.	CURRENTLY AUTHORIZED POWER LEVEL (MWt):3323 MAX	DEPEND CAPAC	ITY
	(MWe-Net): 1036 DESIGN ELECTRICAL R	ATING (MWe-	Net):1078	
3.	POWER LEVEL TO WHICH RESTRICTED (IF A	NY) (MWe-Ne	et): <u>N/A</u>	
4.	REASONS FOR RESTRICTION (IF ANY): N/A			
		THIS MONTH	YR TO DATE	CUMULATIVE
5	NUMBER OF HOURS REACTOR WAS CRITICAL	739.8	3042.0	9323
6.	REACTOR RESERVE SHUTDOWN HOURS	4.2	102.9	1269
7.	HOURS GENERATOR ON LINE	739.8	2969.2	9025
8.	UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
9.	GROSS THERMAL ENERGY GENERATED (MWH)	2184310	8433887	25257176
10.	GROSS ELEC. ENERGY GENERATED (MWH)	724455	27887650	8259408
11.	NET ELEC. ENERGY GENERATED (MWH)	703230	2687914	7882976
12.	REACTOR SERVICE FACTOR	99.4%	83.4%	75.0%
13.	REACTOR AVAILABILITY FACTOR	100%	86.2%	85.2%
14.	UNIT SERVICE FACTOR	99.4%	81.4%	72.6%
15.	UNIT AVAILABILITY FACTOR	99.4%	81.4%	72.6%
16.	UNIT CAPACITY FACTOR (USING MDC)	91.2%	71.1%	61.2%
17.	UNIT CAPACITY FACTOR (USING DESIGN			
	MWe)	87.7%	68.4%	58.8%
18.	UNIT FORCED OUTAGE RATE	0.6%	12.2%	13.9%
19	SHUTDOWNS SCHEDULED OVER NEXT 6 MONTH	IS (TYPE, D)	TE. AND DURA	TION OF EACH

 SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TIPE, DATE, AND DURATION OF EACH) Unit one is scheduled for a refueling, maintenance, modification, and surveillance outage beginning September 3, 1985 and lasting 26 weeks.
 IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: June 9,

1985

* 2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO:	050-373
UNIT:	LASALLE ONE
DATE:	June 10, 1985
COMPLETED BY:	Richard J. Rohrer
TELEPHONE:	(815) 357-6761
	DOCKET NO: UNIT: DATE: COMPLETED BY: TELEPHONE:

MONTH: MAY, 1985 DAY AVERAGE DAILY POWER LEVEL DAY AVERAGE DAILY POWER LEVEL (MWe-Net)

(MWe-Net)

1	924	17	1043	-
2	1040	18	1032	
3	1042	19	766	
4	975	20	915	
5	875	21	879	
6	943	22	905	-
7	1032	23	904	
8	1034	24	878	
9	1036	25	877	
10	1030	26	876	_
11	926	27	873	
12	1030	28	874	
13	1035	29	876	
14	1025	30	883	
15	1020	31	725	
16	1029			

3. UNIT SHUTDOWNS AND POWER REDUCTIONS ATTACHMENT B

REPORT MONTH MAY 1985

COMPLETED BY Richard J. Rohrer TELEPHONE (815)357-6761 DOCKET NO. 050-373 UNIT NAME LaSalle One DATE JUNE 10, 1985

CORRECTIVE ACTIONS/COMMENTS	Power reduced to allow maintenance of heater drains.	Manually scrammed following flooding of Lake Screen House.
METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	1	2
REASON	æ	Α.
DURATION (HOURS)	0.00	4.25
TYPE F: FORCED S: SCHEDULED	S	ы
DATE	850519	850531
NO.	6	10

DOCUMENT 0044r/0005r

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E. UNIQUE REPORTING REQUIREMENTS

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DATE	VALVES	NO & TYPE	PLANT	DESCRIPTION
	ACTUATED	ACTUATION	CONDITION	OF EVENT
5-31-85	1821-F013A 1821-F013B 1821-F013C 1821-F013D 1821-F013E 1821-F013F 1821-F013G 1821-F013H 1821-F013K 1821-F013L	1 Manual 1 Manual 1 Manual 1 Manual 1 Manual 1 Manual 1 Manual 1 Manual 1 Manual	620 psig 545 psig 530 psig 595 psig 510 psig 460 psig 435 psig 435 psig 560 psig 275 psig	Manually opened successively to provide controlled cool-down of the reactor following Main Steam Line Isolation.

1. Safety/Relief valve operations for Unit One.

Valve 1B21-F013L opened at 2330 on May 31, 1985 and closed at 0030 on June 1, 1985. Other valves (1B21-F013 M,N,P,Q,R,S) were opened and will be reported in the Monthly Report for June.

2. ECCS Systems Outages

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The following outages were taken on ECCS Systems during the reporting period.

OUTAGE NO.	EQUIPMENT	PURPOSE OF OUTAGE
0-323-85	0 Diesel Generator Fuel Oil Transfer Pump.	Relocate Cables
1-399-85	lA RHR Service Water Pump.	Repair.
1-400-85	lB RHR Service Water Discharge Valve.	Prevent Pump Runout.
1-425-85	1B Diesel Generator.	Change Turbo filter.
1-447-85	lA RHR Service Water Process Radiation Monitor.	Repair Seal Leak.
1-448-85	High Pressure Core Spray Full-Flow Test Valve.	Repair Leak.
1-449-85	1A RHR Process Radiation Monitor.	Work on Pump Motor.

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3. Off-Site Dose Calculation Manual

Revision 11 was made to the Off-site Dose Calculation Manual, incorporating changes requested by the Nuclear Regulatory Commission.

4. Radioactive Waste Treatment Systems.

There were no significant changes to the radioactive waste treatment system during this reporting period.

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

MAY 1985

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

DOCUMENT ID 0036r/0005r

I. INTRODUCTION

- II. MONTHLY REPORT FOR UNIT TWO
 - A. Summary of Operating Experience
 - B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY RELATED MAINTENANCE
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 - Facility or Procedure Changes Requiring NRC Approval
 - 3. Tests and Experiments Requiring NRC Approval
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 - 1. Safety/Relief Valve Operations
 - 2. ECCS System Outages
 - 3. Off-Site Dose Calculation Manual Changes
 - Major Changes to Radioactive Waste Treatment System

INTRODUCTION

1.

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to an artificial cooling pond using the Illinois River for make-up and blowdown. The architecht-engineer was Sargent and Lundy, and the primary construction contractor was Commonwealth Edison Company.

Unit two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984, and commercial power operation was commenced on June 19, 1984.

This report was compiled by Richard J. Rohrer, telephone number (815)357-6761 extension 575.

MONTHLY REPORT FOR UNIT TWO

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SUMMARY OF OPERATING EXPERIENCE FOR UNIT TWO

MAY 1-31

The reactor was subcritical, and the generator was off-line for the entire month of MAY. Unit Two is in a scheddled outage for maintenance, testing, and modifications. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

- Amendments to facility license or Technical Specifications. Three ammendments were made to the Unit two operating license and Technical Specifications during May. Ammendment 9 modifies the calculational technique used in determining duct heater performance for the standby Gas Treatment System. Ammendment 10 changes the channel check requirements to accommodate new instruments installed by Environmental Qualification modifications. Ammendment 11 incorporates the requirements of 10CFR50.73.
- Facility or procedure changes requiring NRC approval. There were no facility or procedure changes requiring NRC approval during the reporting period.
- Tests and experiments requiring NRC approval.
 There were no tests or experiments requiring NRC approval during the reporting period.
- 4. Corrective Maintenance of Safety Related Equipment. The following table (Table 1) presents a summary of safety-related maintenance completed on Unit Two during the reporting period. The headings indicated in this summary include: Work Request number, Component Name, cause of malfunction, results and effects on safe operation, and corrective action.

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CORRECTIVE MAINTENANCE OF SAFETY RELATED EQUIPMENT

HORE PROVIEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS	CORRECTIVE ACTION ON SAFE OPERATION
WORK REFORM		pilot valve	Potential to cause failure to	Changed "O-ring".
L40557	HCU Accumulator for CRD 42-03.	cap nut.	scram this rod if combined with other events.	
		in the op limit	No indication of valve's positio	n. Cleaned and adjusted
L41791	Main steam line out-	Dirty Contacts on Timic switch.		Timit switch
	"a" pup Suction Valve.	Torque switches at wrong	Valve tripped its thermals when cycled.	Adjusted Torque and limit switches.
L44542	A KIIK SUCCESSI	setting.	cierce close from	Replaced torque switch.
144755	Steam line downstream	Faulty torque switch.	remote handswitch.	
644155	drain.	the and bloom	n Continuity Meter would not	Corrected wiring and
L46397	"4B" Standby Liquid Control Explosive	Wired incorrectly and brown fuse.	function properly.	Teplaced Tall
	Valve Continuity Alarm	I.	Incorrect reading.	Recalibrated Monitor.
L47010	Division I Post-	Out-of-Calibration.		Recalibrated Monitor.
	LOCA OXYGEN HONESS	Out-of-calibration.	Incorrect Reading.	Kecaliblaced Honster
L47011	LOCA Oxygen Monitor.		norformance	Replaced faulty cell.
12020	Division III Batterie	s. Faulty cell.	Degraded battery performance.	Tocreased spring tension
L47273	DIVISION THE	Less-than-desirable spring	g Valve failed local leak rate test.	Increased of any
L47530	Drywell Equipment Drain Inboard	tension.		
	Isolation Valve.	to look coring	Valve failed local leak rate	Increased spring tension
L47578	Drywell Equipment Drain Sump Valve.	Less-than-desired spring tension.	test.	

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CORRECTIVE MAINTENANCE OF SAFETY RELATED BOULPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS	CORRECTIVE ACTION ON SAFE OPERATION
L47708	"A" Standby Liquid Control Explosive Valve Continuity Meter.	Faulty Relay.	Meter falsely indicated complete circuit.	Replaced relay.
L47773	RCIC Ambient Temperature Element.	Cables mislabeled and landed incorrectly.	Instrument would not respond.	Relabeled cables and landed correctly.
L47839	"B" Primary Contain- ment Chill Water Supply Inboard Isolation Valve.	Loose contacts on torque switch.	Valve would not open from remote handswitch.	Tightened torque switch contacts.
L48149	125 Volt DC Battery Charger.	Blown fuses.	No Voltage indication in control room.	Replaced fuses.
L48329	Outboard Feed Water Check Valve "A".	Actuating cylinder was binding.	Would not pull valve to closed position.	Cleaned and lubricated actuator cylinder.
L48375	Scram Discharge Volume Vent Valve.	Air leak on supply to valve actuator.	Possibility to increase the the length of time required to insert rods during a scram if combined with other events.	Cleaned coupling in air line.
L48505	Average Power Range Monitor "F".	Blown fuse.	Half of downscale lights were out.	Replaced fuse.
L48628	Reactor Water Clen-up Inboard Isolation Valve.	Broken torque switch.	Valve would not completely close with motor operator.	Replaced torgue switch.

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CORRECTIVE MAINTENANCE OF SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS	CORRECTIVE ACTION ON SAFE OPERATION
L48654	Standby Gas Treatment System Deluge Valves.	Valves software degraded.	Valves leaked by, causing char- coal filter to get wet.	Replaced software in valves.
L48745	Reactor Building Ventilation Exhaust Damper.	Stuck open contactor.	Damper tripped thermals when cycling open. Burned out the the motor.	Replaced motor, open contactor, and mechanical relay.
L49053	Diesel Generator "2B"	Faulty K33 relay.	Diesel Generator tripped on low oil pressure shortly after starting.	Replaced relay K33.
L49113	LPCS Minimum Flow Switch.	Switch out-of-calibration.	Minimum flow valve did not automatically open when required	Recalibrated switch.

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The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit Two, occurring during the reporting period, May 1 through May 31, 1985. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

Licensee Event Report Number	Date	Title of Occurrence
85-019-00	4-12-85	Primary Containment Group I Isolation Signal.
85-020-00	4-24-85	Missed Service Water Sample.
85-021-00	5-3-85	RCIC Temperature Leak Detection Miswired.
85-022-00	5-16-85	Reactor Scram.

D. DATA TABULATIONS

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The following data tabulations are presented in this report:

- 1. Operating Data Report
- 2. Average Daily Unit Power Level
- 3. Unit Shutdowns and Power Reductions

DOCKET NO. 050-374 UNIT LaSalle Two DATE June 10, 1985 COMPLETED BY Richard J. Rohrer TELEPHONE (815)357-6761

OPERATING STATUS

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1.	REPORTING PERIOD: May, 1985 GROSS H	OURS IN REP	PORTING PERIO	D: 744
2.	CURRENTLY AUTHORIZED POWER LEVEL (MWt	:3323 MAX	DEPEND CAPAC	ITY
	(MWe-Net): 1036 DESIGN ELECTRICAL R	ATING (MWe-	Net):1078	
3.	POWER LEVEL TO WHICH RESTRICTED (IF A	NY) (MWe-Ne	et): N/A	
4.	REASONS FOR RESTRICTION (IF ANY): N/A			
		THIS MONTH	YR TO DATE	CUMULATIVE
5	NUMBER OF HOURS REACTOR WAS CRITICAL	0.0	1399.8	3011.6
6.	REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	125.3
7.	HOURS GENERATOR ON LINE	0.0	1397.3	2934.7
8.	UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
9.	GROSS THERMAL ENERGY GENERATED (MWH)	0.0	4387385	8894977
10.	GROSS ELEC. ENERGY GENERATED (MWH)	0.0	1460387	2945373
11.	NET ELEC. ENERGY GENERATED (MWH)	-9397	1382193	2774510
12.	REACTOR SERVICE FACTOR	0.0%	38.4%	55.5%
13.	REACTOR AVAILABILITY FACTOR	0.0%	38.4%	57.8%
14.	UNIT SERVICE FACTOR	0.0%	38.3%	54.1%
15.	UNIT AVAILABILITY FACTOR	0.0%	38.3%	54.1%
16.	UNIT CAPACITY FACTOR (USING MDC)	-1.2%	36.6%	49.4%
17.	UNIT CAPACITY FACTOR (USING DESIGN			
	MWe)	-1.2%	35.2%	47.5%
18.	UNIT FORCED OUTAGE RATE	0.0%	0.0%	4.4%
19.	SHUTDOWNS SCHEDULED OVER NEXT 6 MONTH	S (TYPE, DA	TE. AND DURA	TION OF FACH

An outage for maintenance and surveillance was begun at 0520 on February 28, 1985.

20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP_June 14, 1985

DOCKET NO:	050-374
UNIT:	LASALLE TWO
DATE:	June 10, 1985
COMPLETED BY:	Richard J. Rohrer
TELEPHONE:	(815) 357-6761
MONTH:	May 1985

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(MWe-Net)

DAY AVERAGE DAILY POWER LEVEL DAY AVERAGE DAILY POWER LEVEL (MWe-Net)

1	-11	17
2	-11	18
3	-11	19
4	-11	20
5	-11	21
6	-11	22
7	-11	23
8	-11	24
9	-11	25
10	-11	26
11	-11	27
12	-12	28
13	-13	29
14	-13	30
15	-13	31
16	-13	

17	-14
18	-17
19	-15
20	-15
21	-14
22	-12
23	-12
24	-12
25	-13
26	-14
27	-15
28	-14
29	-14
30	-14
31	-13

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ATTACHMENT E 3. UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH MAY 1985

DOCKET NO. 050-374 UNIT NAME LaSalle Two DATE JUNE 10, 1985 COMPLETED BY Richard J. Rohrer TELEPHONE (815)357-6761

NO.	DATE	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
3	850228	S	744.00	В	2	Maintenance and Surveillance Outage begun 2-28-85 continues

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief Valve Operations for Unit Two.

DATE	VALVES	NO & TYPE	PLANT	DESCRIPTION
	ACTUATED	ACTUATIONS	CONDITION	OF EVENT
5-23-85	2B21-F013D	l Automatic	Cold Shutdown	Spurious Actuation, cause unknown.

DOCUMENT ID 0036r/0005r

* * 2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

OUTAGE NO.	EQUIPMENT	PURPOSE OF OUTAGE
2-702-85	2E12-F004A	Minor Maintenance.
2-706-85	2B Diesel Generator Fuel Oil Transfer Valve.	Assist Mechanical. Maintenance Department.
2-707-85	Low Pressure Core Spray.	Check Coupling.
2-708-85	2A Diesel Generator "B" Air Compressor.	Repair A/C Poppet relief.
2-710-85	2B RHR Service Water Pump	Remove Motor.
2-714-85	Low Pressure Core Spray Water Leg Fump.	Clean and Lubricate Coupling.
2-740-85	High Pressure Core Spray Spool Piece.	Remove Spool piece.
2-747-85	2B Diesel Generator Fuel Oil Transfer Valve.	Repair Leak.
2-748-85	2E12-C300C.	Repair Motor.
2-755-85	2E12-F050A	Assist Mechanical Maintenance Department.
2-756-85	2E12-F050A	Reapair Binding.
2-760-85	2E12-F064A	Repack Valve.
2-761-85	"A" RHR Pump Breaker.	Install Rubber Boots.
2-763-85	RCIC Pump.	Repair Drain Line.
2-772-85	"B" RHR Service Water Process Radiation Monitor.	Decontamination.
2-774-85	2B Diesel Generator.	Maintenance and Surveillance.
2-785-85	RHR-Shutdown Cooling.	Surveillance.
2-792-85	2A RHR Pump.	Check Oil Level.
2-793-85	2A RHR Pump.	Modification to Service Water Process Radiation Monitor.

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OUTAGE NO.	EQUIPMENT	PURPOSE OF OUTAGE
2-794-85	2A RHR.	Surveillance.
2-799-85	200014	Troubleshooting.
2-801-85	2A RHR Service Water Process	Decontamination.
	Radiation Monitor.	
2-802-85	2E22-F038.	Testing.
2-803-85	"A" LPCI and LPCS Manual	
	Injection Valves.	Surveillance.
2-805-85	High Pressure Core Spray.	Isolate System.
2-806-85	28 Diesel Generator.	Repair Immersion Heater.
2-808-85	High Pressure Core Spray	Determinate.
	Full-Flow Line Heat Trace.	
2-810-85	LPCS Water Leg Pump.	Repair Check Valve.
2-815-85	Division I Water Leg Pump.	Repair Discharge Valve.
2-816-85	2A RHR Service Water Process	Relocate.
	Radiation Monitor Skid.	
2-822-85	High Pressure Core Spray Pump.	Check Breaker.

• * * 3. Off-Site Dose Calculation Manual

Revision 11 was made to the Off-site Dose Calcualtion Manual, incorporating changes requested by the Nuclear Regulatory Commission.

 Radioactive Waste Treatment Systems. There were no changes to the radioactive waste treatment system during this reporting period.



Commonwealth Edison LaSalle County Nuclear Station Rural Route #1, Box 220 Marseilles, Illinois 61341 Telephone 815/357-6761

June 10, 1985

Director, Office of Management Information and Program Control United States Nuclear Regulatory Commission Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering LaSalle County Nuclear Power Station for the period covering May 1 through May 31, 1985.

Very truly yours,

Judwich

G,/J. Diederich Station Manager LaSalle County Station

GJD/RJR/crh

Enclosure

xc: J. G. Keppler, NRC, Region III NRC Resident Inspector LaSalle Gary Wright, Ill. Dept. of Nuclear Safety D. P. Galle, CECO D. L. Farrar, CECO INPO Records Center Ron A. Johnson, PIP Coordinator SNED J. E. Ellis, GE Resident J. M. Nowicki, Asst. Comptroller H. E. Bliss, Nuclear Fuel Services Manager C. F. Dillon, Senior Financial Coordinator

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