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NRC Form 386A (9-83)	LICENSEE EVENT REPORT (LER) TEXT CONTINUATION APPROVED OMB NO. 3150-0104 EXPIRES 8/31/85								104		
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

As part of the work scheduled for completion during the Unit's first refueling outage, Induction Heating for Stress Improvement (IHSI) treatment of the weld at the pipe tee where the Reactor Water Cleanup (RWCU) system (EIIS Code: CE) suction line taps off of the reactor recirculation (EIIS Code: AD) loop 'A' commenced on May 5, 1985. (IHSI is a process which produces a predetermined temperature difference across a pipe wall by inductively heating the outer surface of a pipe weld joint, while simultaneously cooling the inner surface with flowing water. This technique produces compressive residual stresses on the inside surface, thereby reducing the probability of intergranular stress corrosion cracking.) The IHSI procedure for this weld specified that only the reactor recirculation loop 'A' be in service to provide cooling for the inner surface of the weld. RWCU was placed in service by Operations personnel as a precautionary measure. At 0150 on May 5, 1985, the RWCU containment outboard isolation valve closed on a high flow signal. (Unanticipated actuation of an Engineered Safety Feature.) System flow at that time was below the setpoint of the differential pressure switch which provides high flow isolation. However, the switch is calibrated for normal system temperature while in Operational Condition 1 (Run). The difference in water density which results in a higher differential pressure when the system is running cold versus when the system is at normal operating temperature accounts for the switch's actuation at a lower flow rate. The heat was removed from the weld using recirculation flow and RWCU was returned to service. An evaluation of RWCU operation indicates that the system operated as designed during this occurrence. Since the IHSI treatment of the weld is a once-and-done evolution, there is no action required to prevent a similar event.



Pennsylvania Power & Light Company

Two North Ninth Street • Allentown, PA 18101 • 215 / 770-5151

June 3, 1985

U.S. Nuclear Regulatory Commission Document Control Desk Washington, DC 20555

SUSQUEHANNA STEAM ELECTRIC STATION LICENSEE EVENT REPORT 85-017-00 ER 100450 FILE 841-23 PLAS- 085

Docket No. 50-387 License No. NPF-14

Attached is Licensee Event Report 85-017-00. This event was determined reportable per 10CFR50.73(a)(2)(iv), in that the Unit experienced the unanticipated actuation of an engineered safety feature when the reactor water cleanup system containment outboard isolation valve closed on a high flow signal.

AN Thompson for

H.W. Keiser Superintendent of Plant-Susquehanna

LAK/pjg

cc: Dr. Thomas E. Murley Regional Administrator, Region I U.S. Nuclear Regulatory Commission 631 Park Avenue King of Prussia, PA 19406

> Mr. R.H. Jacobs Senior Resident Inspector U.S. Nuclear Regulatory Commission P.O. Box 52 Shickshinny, PA 18655

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