

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Wolf Creek Generating Station	DOCKET NUMBER (2) 0 5 0 0 0 4 8 2	PAGE (3) 1 OF 0 2
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TITLE (4)
ESF Actuation - Control Room Ventilation Isolation

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0	5	0	8	5	8	5	0	1	0	0	0
0	5	0	8	5	8	5	0	1	0	0	0

OPERATING MODE (9) 2	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)									
POWER LEVEL (10) 0 1 0 1 0	20.402(b)	20.408(c)	<input checked="" type="checkbox"/>	50.73(a)(2)(iv)	73.71(a)					
	20.406(a)(1)(i)	50.38(c)(1)	<input type="checkbox"/>	50.73(a)(2)(v)	73.71(c)					
	20.406(a)(1)(ii)	50.38(c)(2)	<input type="checkbox"/>	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 356A)					
	20.406(a)(1)(iii)	50.73(a)(2)(i)	<input type="checkbox"/>	50.73(a)(2)(viii)(A)						
	20.406(a)(1)(iv)	50.73(a)(2)(ii)	<input type="checkbox"/>	50.73(a)(2)(viii)(B)						
20.406(a)(1)(v)	50.73(a)(2)(iii)	<input type="checkbox"/>	50.73(a)(2)(ix)							

LICENSEE CONTACT FOR THIS LER (12)

NAME Merlin G. Williams - Superintendent of Regulatory, Quality and Administrative Services	TELEPHONE NUMBER 3 1 6 3 6 4 - 8 8 3 1
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
B	J	E R I E I	G I O I 6 1 3	N					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On six different occasions, an Engineered Safety Features Actuation Signal (ESFAS) was initiated by a control room intake radiation monitor spurious alarm causing a Control Room Ventilation Isolation Signal (CRVIS). The incidents occurred at 0155 and 1650 CDT on May 6, 1985, 1054 CDT on May 15, 1985, 1037 CDT on May 23, 1985, 1814 CDT on May 26, 1985, and 1407 CDT on May 30, 1985. All required engineered safety features equipment responded properly on each occasion.

The plant was in Mode 3, Hot Standby, prior to initial criticality at the time of the first three incidents. During the May 23, 1985 and May 30, 1985 occurrences, the plant was in Mode 2 with the reactor critical at a power level of 10E-8 amps Intermediate Range. On May 26, 1985, the plant was in Mode 3, Hot Standby, with a Source Range indication of 25 counts/sec(cps). All incidents occurred with the Reactor Coolant System at normal operating pressure and temperature.

These events posed no threat to public health or safety.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Wolf Creek Generating Station	DOCKET NUMBER (2) 81501010482	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		85	013	0	02	OF	02

TEXT (if more space is required, use additional NRC Form 206A or 117)

On six different occasions, an Engineered Safety Features Actuation Signal (ESFAS) was initiated by a spurious electronic "spike" in a control room intake radiation monitor (GK-RE-04). The incidents occurred at 0155 and 1650 CDT on May 6, 1985, 1054 CDT on May 15, 1985, 1037 CDT on May 23, 1985, 1814 CDT on May 26, 1985, and 1407 CDT on May 30, 1985. Each of these spikes resulted in a Control Room Ventilation Isolation Signal (CRVIS), in which all required engineered safety features equipment responded properly.

The plant was in Mode 3, Hot Standby, prior to initial criticality at the time of the first three incidents. During the May 23, 1985 and May 30, 1985 occurrences, the plant was in Mode 2 with the reactor critical at a power level of 10E-8 amps Intermediate Range. On May 26, 1985, the plant was in Mode 3, Hot Standby, with a Source Range indication of 25 counts/sec(cps). All incidents occurred with the Reactor Coolant System at normal operating pressure and temperature.

In each instance, no radiation above normal background was present, as determined by redundant radiation monitor GK-RE-05, and the actuated systems were restored to a normal configuration per plant procedures.

Subsequent investigation of each incident identified a mismatch between the software and hardware in the RM-80 microprocessing unit for the radiation monitor as the probable cause of the spurious alarms. A previous engineering evaluation based on similar problems at another plant concluded that radiation monitors of this type were susceptible to this problem, although it had not occurred previously at Wolf Creek. Resolution of the mismatch for the radiation monitors is expected from the manufacturer in the near future and will be installed when it becomes available.

These events posed no threat to the public health or safety.

Previous occurrences: None



KANSAS GAS AND ELECTRIC COMPANY

GLENN L. KOESTER
VICE PRESIDENT - NUCLEAR

May 31, 1985

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Mr. R.P. Denise, Director
Wolf Creek Task Force
U.S. Nuclear Regulatory Commission
Region IV
611 Ryan Plaza Drive, Suite 1000
Arlington, Texas 76011

KMLNRC 85-144
Re: Docket No. STN 50-482
Subj: Licensee Event Report 85-013-00

Gentlemen:

The enclosed Licensee Event Report is submitted pursuant to 10 CFR 50.73 (a) (2) (iv) concerning an Engineered Safety Feature actuation.

If you have any questions concerning this matter, please contact me or Mr. Otto Maynard of my staff.

Yours very truly,

Glenn L. Koester
Vice President - Nuclear

GLK:dab

Enclosure

xc: PO'Connor (2), w/a
JCummins, w/a

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