



231 W Michigan, PO Box 2046, Milwaukee, WI 53201

(414) 221-2345

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10 CFR 50, Appendix J

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U.S. NUCLEAR REGULATORY COMMISSION
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Gentlemen:

DOCKET 50-301
REACTOR CONTAINMENT BUILDING INTEGRATED LEAK RATE TEST
POINT BEACH NUCLEAR PLANT, UNIT 2

In accordance with Section 15.4.4.I.D of the Point Beach Nuclear Plant Technical Specifications and Section V.B. of 10 CFR 50, Appendix J, this technical report is submitted summarizing the Type A containment integrated leakage rate test conducted during our 1992 Unit 2 refueling outage. Also included are summaries of the results from the Type B and C leak tests conducted subsequent to our 1989 Type A test. The Type B and C leak tests were performed during the refueling outages of September-November 1989, October-November 1990, and September-November 1991. The 1992 Type A test was performed by Wisconsin Electric with assistance from Bechtel Power Corporation. We have enclosed a summary report prepared in accordance with the format specified in ANSI/ANS 56.8 - 1987. The complete documentation (which includes all procedures, equipment calibrations, test data, and results) will be retained by Wisconsin Electric and is available for NRC review at Point Beach Nuclear Plant.

Our 1992 Unit 2 Type A integrated leak rate test was conducted in accordance with 10 CFR 50, Appendix J, Point Beach Nuclear Plant Operations Refueling Tests (ORT)-17, "Containment Integrated Leak Rate Test, Unit 2," and Bechtel Corporation Topical Report (BN-TOP-1), "Testing Criteria For Integrated Leakage Rate Testing of Containment Structures For Nuclear Power Plants." Our Type B and Type C penetration tests were conducted in accordance with 10 CFR 50, Appendix J and various Point Beach Nuclear Plant Operations Refueling Tests.

The overall as-found total time leakage rate measured for the 1992 Type A test was 0.158%/day at the 95% upper confidence level. This is 79% of the operational leakage rate allowed by Point Beach Nuclear Plant Technical Specifications. The leakage rate reported

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includes a 0.007%/day penalty for non-standard valve alignments, containment net free volume changes, and pressurized component additions to the containment.

The overall as-found leakage of the Type B and C tests conducted during the 1989 refueling outage was 18,261 standard cubic centimeters per minute (sccm) or 8% of 0.6L_s. After repairs, the overall leakage rate was 17,553 sccm or 8% of 0.6L_s (0.6L_s is 60% of the maximum allowable leakage as allowed by Point Beach Nuclear Plant Technical Specifications).

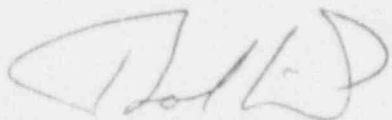
The overall as-found leakage of the Type B and C tests conducted during the 1990 refueling outage was 26,512 sccm or 11% of 0.6L_s. After repairs, the overall leakage rate was 21,973 sccm or 10% of 0.6L_s.

The overall as-found leakage of the Type B and C tests conducted during the 1991 refueling outage was 39,095 sccm or 17% of 0.6L_s. After repairs, the overall leakage rate was 21,088 sccm or 9% of 0.6L_s.

The results of all Type A, Type B, and Type C tests performed subsequent to our 1989 Unit 2 outage, meet or exceed the requirements of Point Beach Nuclear Plant Technical Specifications and Section III of 10 CFR 50, Appendix J.

If you have any questions concerning this report or require additional information, please feel free to contact us.

Sincerely,



Bob Link
Vice President
Nuclear Power

Enclosures

cc: NRC Resident Inspector
NRC Regional Administrator, Region III

DMC/jg

SUMMARY REPORT
REACTOR CONTAINMENT BUILDING INTEGRATED LEAK RATE TEST
POINT BEACH NUCLEAR PLANT UNIT 2, 1992

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REPORT OF REACTOR CONTAINMENT BUILDING INTEGRATED
LEAKAGE RATE TEST FOR POINT BEACH UNIT 2

1.0 GENERAL DATA

- 1.1 Owner: Wisconsin Electric Power Company
- 1.2 Docket Number: 50-301
- 1.3 Location: Two Rivers, Wisconsin
- 1.4 Containment Description: 3½ foot pre-stressed post-tensioned concrete cylinder with ½ inch welded ASTM A-442 steel liner
- 1.5 Date Test Was Completed: October 1, 1992

2.0 TECHNICAL DATA

- 2.1 Containment Net Free Volume: 1,000,000 cubic feet
- 2.2 Design Pressure: 60 psig
- 2.3 Design Temperature: 286°F
- 2.4 Calculated Accident Peak Pressure (Pac): 53 psig
- 2.5 Calculated Accident Peak Temperature (Tac): 278°F

3.0 TEST DATA

- 3.1 Test Method: Absolute
- 3.2 Data Analysis Technique: Total Time
- 3.3 Test Pressure: 31 psig
- 3.4 Maximum Allowable Leakage Rate, L_1 : 0.400%/day
- 3.5 Reduced Pressure Allowable Leak Rate, L_1 : 0.268%/day
- 3.6 Allowable Operational Leak Rate, .75 L_1 : 0.201%/day
- 3.7 Integrated Leakage Rate Test Results
 - 3.7.1 Total Time Leak Rate 0.088%/day
 - 3.7.2 95% Upper Confidence Level 0.151%/day

3.8 Verification Test Results

3.8.1	Imposed Verification Leak Rate	0.268%/day
3.8.2	Total Time Analysis	0.319%/day
3.8.3	Verification Test Limits	
	Lower Limit	0.289%/day
	Upper Limit	0.423%/day

3.9 Penalties

Containment Net Volume Changes	0.006%/day
Penetrations Not Aligned	0.000%/day
Pressurized Component Additions	0.001%/day

Total 0.007%/day

3.10 As-Found Leak Rate	0.158%/day
Corrected for penalties at 95% upper confidence level	

4.0 ANALYSIS AND INTERPRETATION

The sixth periodic Type A integrated leakage rate test (ILRT) of the Point Beach Nuclear Plant Unit 2 containment was performed September 27 to October 1, 1992, with satisfactory results. The testing program was conducted in accordance with the requirements of the Point Beach Technical Specifications, Section 15.4.4, "Containment Tests" and Procedure ORT-17, "Containment Integrated Leak Rate Test."

Eight penetrations, listed in Section 5.0 of this report, were not in a post-LOCA alignment during the ILRT. This required additions, based on minimum pathway leakage analysis, to the ILRT result of 0.0002%/day.

The pressurizer, reactor coolant drain tank, pressurizer relief tank, and Containment Sump A were all vented to the containment during the ILRT. Water level changes in these tanks resulted in a decrease in the containment net free volume of 6.52 ± 13.70 cubic feet during the test. The correction to the ILRT result for this net volume change is 0.0060%/day.

The accumulators for the inner purge supply and purge exhaust valve seals, and the safety injection accumulators were sources for pressurized component additions to the containment during the ILRT. The total pressurized component additions to the containment were 0.096 lbm/hour. The correction to the ILRT result for this air addition is 0.001%/day.

The total leakage rate correction for penetrations not in post-LOCA alignment, net volume changes, and pressurized component additions is 0.0072%/day.

The calculated leakage rate during the ILRT was 0.088%/day (total time). The calculated 95% upper confidence level was 0.151%/day (total time). Adding the total leakage rate corrections for penetrations not in a post-LOCA alignment, containment net volume changes, and pressurized component additions, yields the corrected leakage rates as follows:

	Total Time Leakage Rate, %/Day	
	<u>Leakage Rate</u>	<u>UCL</u>
Calculated	0.088	0.151
Corrections	0.007	0.007
As-Found/As-Left	0.095	0.158

No valve repairs were performed prior to the ILRT. Therefore, the as-found test is equivalent to the as-left test. Since the corrected 95% upper confidence level for the total time is less than $0.75L_1$ (0.201%/day), the test results demonstrated the leakage through the primary containment and systems and components penetrating primary containment does not exceed the allowable leakage rates specified in the Point Beach Nuclear Plant FSAR and Technical Specifications.

5.0 TYPE C LEAKAGE CORRECTIONS

Due to the test configuration of the containment, 8 penetrations were not subjected to the Type A test pressure. These penetrations, justification, and as-found/as-left Type C test results are identified below:

<u>Penetration</u>	<u>Justification</u>	<u>Type C Results (Minimum Pathway)</u>
CVCS Supply Line (#26)	In use for reactor coolant system purification	27 sccm
CVCS Letdown Line (#10)	In use for reactor coolant system purification	70 sccm
RCP "A" Seal Injection Line (#29A)	Maintain flow of clean water through RCP seals	1 sccm
RCP "B" Seal Injection Line (#29B)	Maintain flow of clean water through RCP seals	1 sccm
Auxiliary Charging Line (#32C)	Proper venting of line is not possible when seal injection lines are in use	0 sccm
PACVS Supply Line (#31C)	Valve line-up modified to provide a flow verification path	0 sccm
Sump A Drain (#71)	Line was not vented in order to maintain the ability to pump Sump A during the ILRT	1 sccm
RCDT Pump Suction (#9)	Line was not vented in order to maintain the ability to pump the RCDT	0 sccm

The 8 penetrations not subjected to Type A test pressure were tested in the as-found condition following the Type A test at reduced pressure. All values are ± 2 sccm.

6.0 TYPE B TESTS PERFORMED BETWEEN REFUELINGS

<u>Procedure</u>	<u>Component</u>	<u>Date</u>	<u>Leak Rate</u>
TS-10	Upper Personnel Hatch	5/5/90	330 sccm
TS-10	Lower Personnel Hatch	5/5/90	965 sccm
TS-36	Purge Exhaust (V1)	5/20/90	403 sccm
TS-36*	Purge Supply (V2)	5/20/90	2240 sccm
TS-36	Purge Supply (V2)	5/29/90	1836 sccm
TS-10	Upper Personnel Hatch	5/1/91	360 sccm
TS-10	Lower Personnel Hatch	5/1/91	312 sccm
TS-36	Purge Exhaust (V1)	5/24/91	177 sccm
TS-36	Purge Supply (V2)	5/24/91	183 sccm
TS-10	Upper Personnel Hatch	4/29/92	1233 sccm
TS-10	Lower Personnel Hatch	4/29/92	1857 sccm
TS-36	Purge Exhaust (V1)	6/13/92	77 sccm
TS-36	Purge Supply (V2)	6/13/92	863 sccm

*IA-876, check valve to boot seal for purge supply valve (V2), was sticking and not permitting sufficient air flow to boot seal. The leakage rate of 2240 sccm for the purge supply valve (V2) is in excess of administrative limits, but less than the limits specified in Technical Specifications Section 15.4.4.III.B. IA-876 was replaced during the following refueling outage (U2R16).

7.0 TYPE C TESTS PERFORMED BETWEEN REFUELINGS

<u>Procedure</u>	<u>Penetration</u>	<u>Date</u>	<u>Valve(s) Tested</u>	<u>Leak Rate</u>
*ORT-61	71	1/10/91	2WL-1723 & 1728	6 sccm
**ORT-36	28b	1/26/91	2SC-966B & 953	2500 sccm

*ORT-61 was performed as a result of adding a new test line to Penetration P-71, Sump 'A' Drain to Aux. Building.

**ORT-36 was performed for post maintenance on SC-966B. The leakage rate of 2500 sccm is in excess of administrative limits, but less than the limits specified in Technical Specifications Section 15.4.4.III.B. SC-966B was replaced during the following outage (U2R17).

8.0 TYPE B AND C TEST SUMMARY

The following tables are the as-found and as-left leakage rates for the Type B and C tests conducted between 1989 and 1992 ILRT's.

OPERATIONS REFUELING TEST REPORT
Short List of Database, Index numbers (1-66)
Unit 2

INDEX	ORT	NAME	PRE-OUT	AS FOUND	POST-OUT	POST-DATE
01	12	Fuel Transfer Tube Flange Seal	113	17	17	11/12/89
02	43H	Containment Sampling Line	3	39	39	10/17/89
03	13	Equipment Hatch Flange Seals	61	79	79	11/10/89
04	44H	PACVS Vent Line	11	19	19	09/25/89
05	TS-10U	El. 66' Personnel Hatch Test	591	6	6	11/19/89
06		SPARE	0	0	0	
07	TS-10L	El. 26' Personnel Hatch Test	1660	1	1	11/18/89
08	46H	Auxiliary Charging Line	2	62	62	10/17/89
09	75	Mechanical Penetration	316	608	608	10/12/89
10	47H	Instrument Air Supply	1001	1070	1070	09/24/89
11	76	Mechanical Penetration	8	2	2	10/12/89
12	48H	Instrument Air Supply	493	435	435	09/24/89
13	77	Mechanical Penetration	5	7	7	10/12/89
14	49	Service Air Supply	55	61	2090	11/20/89
15	78	Mechanical Penetration	141	2	2	10/21/89
16	50H	PRT to Gas Anal. Sample Line	2	4	4	10/10/89
17	79	Mechanical Penetration (Xfer Tube)	0	8	8	10/19/89
18	51	A Steam Generator Sample Isol Valve	68	937	937	10/16/89
19	80	Mechanical Penetration	4	1	1	10/20/89
20	52	B Steam Generator Sample Isol Valve	15	17	17	10/16/89
21	81	Mechanical Penetration	30	0	0	10/11/89
22	53H	RCDT to Gas Anal. Sample Line	3	3	3	10/10/89
23	82	Mechanical Penetration	31	2	2	10/12/89
24	54	B Steam Generator Blowdown (5958)	1087	744	744	10/02/89
25	83	Mechanical Penetration	23	2	2	10/20/89
26	55	A Steam Generator Blowdown (5959)	90	635	635	10/02/89
27	84	Mechanical Penetration	16	1	1	10/19/89
28	56	Heating Steam to Containment	83	25	191	11/09/89
29	25H	RCDT Pump Suction Line	4	4	4	11/14/89
30	57	Containment Condensate Return	3270	2040	31	11/09/89
31	26H	Letdown Line	98	513	513	10/17/89
32	58	Containment Test Connection	87	35	35	10/12/89
33	27H	RCP Seal Return	121	202	202	10/06/89
34	59	862A Spray Discharge Check	21	307	11	10/10/89
35	28	Demineralized Water Line	0	1	1	10/12/89
36	60	862B Spray Discharge Check	1	67	16	10/10/89
37		SPARE	0	0	0	
38	61	Sump 'A' Drain to Aux. Building	5	49	49	10/21/89
39	30H	RCDT & PRT Vent	2	2	2	10/06/89
40	TS-36E	Purge Exhaust	34	280	391	11/19/89
41	31	N ² Supply to Prt	15	12	12	10/05/89
42	TS-36S	Purge Supply	2840	480	4920	11/21/89
43	32	N ² Supply to SI Accumulators	480	730	487	10/01/89
44	64H	RE-211 & RE-212 Supply	205	305	305	10/15/89
45	33H	PACVS Return Line	0	35	35	10/02/89
46	65H	RE-211 & RE-212 Return	319	358	358	10/15/89
47	34	Charging Line Check Valve	525	51	51	10/17/89
48	66T	Containment Pressure	2	10	10	10/17/89
49	35	Hot Leg Sample	136	4	47	10/13/89

POINT BEACH NUCLEAR POWER

OPERATIONS REFUELING TEST REPORT
 Short List of Database, Index numbers (1-66)
 Unit 2

INDEX	ORT	NAME	PRE-OUT	AS FOUND	POST-OUT	POST-DATE
50	67T	CCW to Excess Letdown HX	214	11	11	10/01/89
51	36	Press. Liquid Sample Line	2	3330	131	11/17/89
52	67F	CCW from Excess Letdown HX	14	25	25	10/01/89
53	37	Press. Steam Sample Line	70	230	49	10/07/89
54	68T	CCW to P1A	88	620	620	09/30/89
55		SPARE	0	0	0	
56	68F	CCW from P1A	288	620	620	10/02/89
57	39	No. 1 Seal Injection to P1A	128	34	34	10/01/89
58	69T	CCW to P1B	49	2450	926	10/12/89
59	40	No. 1 Seal Injection to P1B	2	5	5	10/01/89
60	69F	CCW from P1B	33	535	535	09/30/89
61		SPARE	0	0	0	
62	72	Electrical Penetration	6	1	1	10/19/89
63	42H	Reactor Makeup Water Supply	1	122	122	10/12/89
64	85	Eddy Current Cables	0	1	1	11/06/89
65	73	Electrical Penetration	2	11	11	10/21/89
66	71	Electrical Penetration	2	0	0	10/12/89
Grand Total			14976	18261	17553	

POINT BEACH NUCLEAR POWER

OPERATIONS REFUELING TEST REPORT
 Short List of Database, Index numbers (1-66)
 Unit 2

July 21, 1992

INDEX	ORT	NAME	PRE-OUT	AS FOUND	POST-OUT	POST-DATE
01	12	Fuel Transfer Tube Flange Seal	17	34	34	10/28/90
02	43H	Containment Sampling Line	39	4	4	10/12/90
03	13	Equipment Hatch Flange Seals	79	5	5	10/25/90
04	44H	PACVS Vent Line	19	4	4	10/11/90
05	TS-10U	El. 66' Personnel Hatch Test	330	1960	1960	11/11/90
06		SPARE	0	0	0	
07	TS-10L	El. 26' Personnel Hatch Test	965	492	492	11/09/90
08	46H	Auxiliary Charging Line	62	4	4	10/18/90
09	75	Mechanical Penetration	608	1148	1148	10/16/90
10	47H	Instrument Air Supply	1070	1823	1823	10/12/90
11	76	Mechanical Penetration	2	0	0	10/17/90
12	48H	Instrument Air Supply	435	505	505	10/12/90
13	77	Mechanical Penetration	7	0	0	10/17/90
14	49	Service Air Supply	2090	2290	580	11/12/90
15	78	Mechanical Penetration	2	12	12	10/18/90
16	50H	PRT to Gas Anal. Sample Line	4	4	4	10/19/90
17	79	Mechanical Penetration (Xfer Tube)	8	70	70	10/27/90
18	51	A Steam Generator Sample Isol Valve	937	933	933	10/11/90
19	80	Mechanical Penetration	1	9	9	10/17/90
20	52	B Steam Generator Sample Isol Valve	17	8	8	10/11/90
21	81	Mechanical Penetration	0	33	33	10/11/90
22	53H	RCDT to Gas Anal. Sample Line	3	1	1	10/24/90
23	82	Mechanical Penetration	2	1	1	10/13/90
24	54	B Steam Generator Blowdown (5958)	744	1706	1706	10/31/90
25	83	Mechanical Penetration	2	1	1	10/17/90
26	55	A Steam Generator Blowdown (5959)	635	217	217	10/30/90
27	84	Mechanical Penetration	1	2	2	10/17/90
28	56	Heating Steam to Containment	191	311	311	10/13/90
29	25H	RCDT Pump Suction Line	4	312	312	10/31/90
30	57	Containment Condensate Return	31	55	55	11/07/90
31	26H	Letdown Line	513	57	57	10/18/90
32	58	Containment Test Connection	35	34	34	10/19/90
33	27H	RCP Seal Return	202	150	150	10/21/90
34	59	862A Spray Discharge Check	11	1207	1785	11/07/90
35	28	Demineralized Water Line	1	76	76	10/21/90
36	60	862B Spray Discharge Check	16	489	534	10/10/90
37		SPARE	0	0	0	
38	61	Sump 'A' Drain to Aux. Building	49	4	4	11/02/90
39	30H	RCDT & PRT Vent	2	1	1	10/22/90
40	TS-36E	Purge Exhaust	403	310	648	11/11/90
41	31	N ² Supply to PRT	12	32	32	10/19/90
42	TS-36S	Purge Supply	1836	1360	2440	11/11/90
43	32	N ² Supply to SI Accumulators	487	257	257	10/12/90
44	64H	RE-211 & RE-212 Supply	305	612	612	10/12/90
45	33H	PACVS Return Line	35	4	4	10/11/90
46	65H	RE-211 & RE-212 Return	358	400	400	10/12/90
47	34	Charging Line Check Valve	51	34	11	11/04/90
48	66T	Containment Pressure	10	35	35	10/12/90
49	35	Hot Leg Sample	47	126	4	11/07/90

POINT BEACH NUCLEAR POWER

OPERATIONS REFUELING TEST REPORT
 Short List of Database, Index numbers (1-66)
 Unit 2

July 21, 1992

INDEX	ORT	NAME	PRE-OUT	AS FOUND	POST-OUT	POST-DATE
50	67T	CCW to Excess Letdown HX	11	1	1	10/13/90
51	36	Press. Liquid Sample Line	131	224	1402	11/06/90
52	67F	CCW from Excess Letdown HX	25	1	1	10/13/90
53	37	Press. Steam Sample Line	49	3140	588	11/07/90
54	68T	CCW to P1A	620	103	103	10/09/90
55		SPARE	0	0	0	
56	68F	CCW from P1A	620	1330	1330	10/09/90
57	39H	No. 1 Seal Injection to P1A	34	1	1	10/17/90
58	69T	CCW to P1B	926	2360	470	11/04/90
59	40H	No. 1 Seal Injection to P1B	5	2	2	10/17/90
60	69F	CCW from P1B	535	2200	470	11/04/90
61		SPARE	0	0	0	
62	72	Electrical Penetration	1	0	0	10/13/90
63	42H	Reactor Makeup Water Supply	122	14	14	10/14/90
64	85	Eddy Current Cables	1	1	270	11/06/90
65	73	Electrical Penetration	11	1	1	10/18/90
66	71	Electrical Penetration	0	2	2	10/18/90
Grand Total			15769	26512	21973	

POINT BEACH NUCLEAR POWER

OPERATIONS REFUELING TEST REPORT
 Short List of Database, Index numbers (1-66)
 Unit 2

July 21, 1992

INDEX	ORT #	NAME	PRE-OUT	ONE-TEST	POST-OUT	POST-DATE
01	12	Fuel Transfer Tube Flange Seal	34	276	276	10/31/91
02	43H	Containment Sampling Line	4	5	5	10/01/91
03	13	Equipment Hatch Flange Seals	5	10	10	10/30/91
04	44H	PACVS Vent Line	4	12	12	10/01/91
05	TS-10U	El. 66' Personnel Hatch Test	360	657	657	11/07/91
06		SPARE	0	0	0	
07	TS-10L	El. 26' Personnel Hatch Test	312	4200	4200	11/07/91
08	46H	Auxiliary Charging Line	4	1	1	10/14/91
09	75	Mechanical Penetration	1148	47	47	10/19/91
10	47H	Instrument Air Supply	1823	1624	1624	10/11/91
11	76	Mechanical Penetration	0	0	0	10/19/91
12	48H	Instrument Air Supply	505	484	317	10/11/91
13	77	Mechanical Penetration	0	13	13	10/19/91
14	49	Service Air Supply	580	80	80	10/28/91
15	78	Mechanical Penetration	12	0	0	10/21/91
16	50H	PRT to Gas Anal. Sample Line	4	3	3	10/18/91
17	79	Mechanical Penetration (Xfer Tube)	70	0	0	10/31/91
18	51	A Steam Generator Sample Isol Valve	933	1627	1627	10/10/91
19	80	Mechanical Penetration	9	2	2	10/08/91
20	52	B Steam Generator Sample Isol Valve	8	27	77	11/08/91
21	81	Mechanical Penetration	13	3	3	10/01/91
22	53H	RCDT to Gas Anal. Sample Line	1	8	8	10/18/91
23	82	Mechanical Penetration	1	4	4	10/22/91
24	54	B Steam Generator Blowdown (5958)	1706	1940	1940	10/19/91
25	83	Mechanical Penetration	1	1	1	10/23/91
26	55	A Steam Generator Blowdown (5959)	217	1813	1813	10/19/91
27	84	Mechanical Penetration	2	5	5	10/23/91
28	56	Heating Steam to Containment	311	3	3	10/01/91
29	25H	RCDT Pump Suction Line	312	27	27	11/01/91
30	57	Containment Condensate Return	55	84	84	10/08/91
31	26H	Letdown Line	5	209	209	10/12/91
32	58	Containment Test Connection	3	22	22	10/22/91
33	27H	RCP Seal Return	140	181	181	10/09/91
34	59	862A Spray Discharge Check	1785	1222	1222	11/04/91
35	28	Demineralized Water Line	76	5	5	11/04/91
36	60	862B Spray Discharge Check	534	138	138	11/04/91
37		SPARE	0	0	0	
38	61	Sump 'A' Drain to Aux. Building	6	1	1	11/03/91
39	30H	RCDT & PRT Vent	1	4	4	10/11/91
40	TS-36E	Purge Exhaust	177	517	517	11/07/91
41	31	N ² Supply to PRT	32	9730	0	10/08/91
42	TS-36S	Purge Supply	183	2610	2610	11/07/91
43	32	N ² Supply to ST Accumulators	357	271	271	09/30/91
44	64H	RE-211 & RE-212 Supply	612	850	850	10/14/91
45	33H	PACVS Return Line	4	7	7	09/30/91
46	65H	RE-211 & RE-212 Return	400	447	447	10/14/91
47	34	Charging Line Check Valve	11	11	11	10/14/91
48	66T	Containment Pressure	35	551	551	10/01/91
49	35	Hot Leg Sample	4	224	224	10/16/91

POINT BEACH NUCLEAR PLANT

OPERATIONS REFUELING TEST REPORT
 Short List of Database, Index numbers (1-66)
 Unit 2

July 21, 1992

INDEX	ORT #	NAME	PRE-OUT	ONE-TEST	POST-OUT	POST-DATE
50	67T	CCW to Excess Letdown HX	1	4	12	11/02/91
51	36	Press. Liquid Sample Line	2500	2522	15	11/09/91
52	67F	CCW from Excess Letdown HX	1	4	4	10/01/91
53	37	Press. Steam Sample Line	588	1420	50	11/03/91
54	68T	CCW to P1A	103	106	177	10/27/91
55		SPARE	0	0	0	
56	68F	CCW from P1A	1330	57	57	10/04/91
57	39H	No. 1 Seal Injection to P1A	1	2	2	10/03/91
58	69T	CCW to P1B	470	3003	397	11/02/91
59	40H	No. 1 Seal Injection to P1B	2	6	6	10/03/91
60	69F	CCW from P1B	470	3	87	11/02/91
61		SPARE	0	0	0	
62	72	Electrical Penetration	0	1	1	10/01/91
63	42H	Reactor Makeup Water Supply	14	30	30	10/22/91
64	85	Eddy Current Cables	270	1972	132	11/05/91
65	73	Electrical Penetration	1	1	1	10/01/91
66	71	Electrical Penetration	2	8	8	10/08/91
Grand Total			18565	39095	21088	

9.0 COMMENTS ON VALVES WITH LEAKAGE APPROACHING OR EXCEEDING 0.6L,
AT STANDARD TEMPERATURE AND PRESSURE

TS 36 (1990)

IA-876, check valve to boot seal for purge supply valve (V2), was sticking and not permitting sufficient air flow to boot seal. The leakage rate of 2240 sccm for the purge supply valve (V2) is in excess of administrative limits, but less than the limits specified in Technical Specifications Section 15.4.4.III.B. IA-876 was replaced during the following refueling outage (U2R16).

ORT 67 (1991)

Check valve, 2CC-767, excess letdown heat exchanger supply, would not seat properly during the initial Type C test. The required test pressure could not be achieved which precluded qualification of the leakage rate. A second test with a higher volume test rig was performed and the check valve seated with a resulting leakage rate of 4 sccm. Maintenance was then performed on the valve to determine the cause for not seating the first time. The valve seat was cleaned and a new O-ring installed. Post-maintenance testing was completed satisfactorily with a leakage rate of 12 sccm. (Licensee Event Report No. 91-004-00, 11/15/91)

ORT 31 (1991)

Diaphragm valve, 2RC-595, nitrogen supply to the pressurizer relief tank, had an as-found leakage rate of 9730 sccm. This leakage rate is in excess of administrative limits, but less than the limits specified in Technical Specifications Section 15.4.4.III.B. The cause of the excess leakage was the result of the stem travel jack nut on the valve being too tight and thus preventing the valve from fully closing. The stem lock nut was adjusted and post-maintenance testing was completed satisfactorily with a leakage rate of 0 sccm. (Not reportable, but covered in Licensee Event Report No. 91-004-00, 11/15/91)

ORT 36 (1991)

Globe valve, 2SC-966B, pressurizer liquid sample line, had an as-found leakage rate of 2520 sccm. This leakage rate is in excess of administrative limits, but less than the limits specified in Technical Specifications Section 15.4.4.III.B. Investigation revealed a crack in the valve body starting below the valve seat and extending to the inlet side of the valve. The valve body was replaced and post-maintenance testing was completed satisfactorily with a leakage rate of 15 sccm. (Not reportable, but covered in Licensee Event Report No. 91-004-00, 11/15/91)