Callaway Plant Post Office Sox 520 Fotmo: Memory 5525



December 7, 1992

U. S. Nuclear Regulatory Commission Document Control Desk Washington, D. C. 20555

**ULNRC-2733** 

Gentlemen:

## DOCKET NUMBER 50-483 CALLAWAY PLANT UNIT 1 FACILITY OPERATING LICENSE NPF-36 VOLUNTARY LICENSEE EVENT REPORT 92-012-00 ESSENTIAL SERVICE WATER VALVE MAINTENANCE ERROR

The enclosed Licensee Event Report is submitted voluntarily to discuss the operability, causes and corrective actions for the 'B' ESW train being declared inoperable and subsequently operable.

WR Campbell

W. R. Campbell Manager, Callaway Plant

WRC/SES/MNF/lrj

Enclosure

cc: Distribution attached

110033

9212110192 921207 PDR ADUCK 05000483 5 PDR

TERR

#### cc distribution for ULNRC-2733

Mr. A. Bert Davis Regional Administrator U. S. Nuclear Regulatory Commission Region III 799 Roosevelt Road Glen Ellyn, IL 60137

Mr. L. Raynard Wharton (2 copies) U. S. Nuclear Regulatory Commission OWFN - Mail Stop 13E21 washington, D. C. 20555

Manager, Electric Department Missouri Public Service Commission P. O. Box 360 Jefferson City, MO 65102

Records Center Institute of Nuclear Power Operations Suite 1500 1100 Circle 75 Parkway Atlanta, GA 30339

Mr. Art Mah Wolf Creek Nuclear Operating Corporation P. O. Box 411 Burlington, KS 66839

Mr. I. N. Jackiw Chief, Project Section 3C U. S. Nuclear Regulatory Commission Region III 799 Roosevelt Road Glen Ellyn, IL 60137

NRC Senior Resident Inspector

Calls	Y NAME (	Plan		nit 1							DOCI	KET NUR	MBER	(2)	0	1.	1.	1		PAGE	(3) OF	010							
TITLE 14	1	Esser	tia	1 Ser	via	e Wa	er \	/alve	Mainta	mance	Error		~ 1	<u> </u>	<u>× 1</u>	-	~		1.0	-	- 1	-	ori	<u> </u>					
		Anader		a orea						ATTALLEA	Laitor																		
EVI	ENT DATE	(6)	1		LE	RNUMBER	(6)		REF	PORT DAT	E (7)				0	THEA	FACI	LITIE	SINV	QLV	ED (8)								
MONTH	DAY	YEAR	YE	RAR		NUMBER	4	REV. NO.	MONTH	DAY	YEAR			FACILITY	( NA	MES					DOCK	ET N	UMBER	S)					
	-		-		+		-									-		-	0	5	0	01	01	11					
0 4	0 9	9 2	9	2 -		0111	2 -	010	112	0 7	9 2								0	5	0	01	0	1.1					
OPI	ERATING ODE (B)	6	174	HS REPO	RT	S SUBMIT	TEO PU	REUANT	TO THE RE	QUAREMEN	TS OF 10	CFR : (C	heck	one or m	tore a	it the	tollow	ingi	(11)										
				20.40	25.4			-	20.4051	9		1	60	73(n)(2)	(iv)				T	73	.71(5)								
PONE	R		-	20.406	61a)	(116)			60.38(c)	(1)			60	73(a)(2)	193				E	172	71(c)								
LEVEL I	(10) 0 0 0 20.406141(100)								50.36(c)	(2)			60	73(a)(2)	(96)				X	10	CHER.	(Spec	ay in						
				20.408	Sie'	11(60)-			50.73(a)	(ASS)		-	50.73(a)(2)(viii)(A)							Ab	stract	belov	and in						
			L	20.408	5(a)	(1)(iv)		-	60.73(a)	(2)(0)			50.73(s)(2)(vsi)(B)							Text, NRC Form 368A)									
				20,408	c (a)	127085			50.72(4)	(2100)	60.73(a)(2)(x)							V	olu	nta	ry								
									LA ENSEE	CONTAC	FOR THIS	THIS LER (12)							and a second										
NAME																-	EACO	OF	TEL	EPH	ONEN	UMB	ER						
C. A. State of the second	T C					Same &	dala d									17	1	1	1.1	É I	11	1		1.1					
Sieve	E. 31	ampse	мı,	supe	TV	180F, 2	ne i	Licens	ang	an sinan inte	and the second second		-	-		3	11	4	6	7	6	-	810	7					
	1				-	MANUFA	C.	REPOR	R EACH CO	DAPONEN	T FAILURE	DESCRI	BED H	N THES	Q.435	RT (1)	3)	Man	UFAC		1 40	PORT	ARIE						
CAUSE	SYSIB	M CC	34tPC	NENT	1	TURER		TO NE	ROS	· - 1	CAUSE	SYSTE	м	COM	PONE	ENT_		TL	IRER		1	ONP	ROS						
	1.1		. 1	1		11	1							1	1	6		1	1										
				1	+		-			1		-	1		-d	-	1		diam's	-	-								
	L.				1	1	1								1	1		1	1	L									
					SU	PPLEMPNT	AL REP	ORT EXPE	CTED (14)								EXPE	CTE	D		MO	VITH	DAY	YEAR					
									h.							SUBMISSE DATE (1)							1	1					

bypass valve was found approximately 25% open when the remote valve position indications indicated closed. This condition existed since 4/12/92 following valve actuator maintenance which adjusted limit switches during Refuel 5. The 'B' ESW train was initially thought to be inoperable, but subsequently was determined operable in its as found position based on an engineering evaluation supported by an analysis completed by the architect engineer. This report is being made voluntarily. The plant was in Mode 6-Refueling at the time of the event.

The root cause of this event is inadequate communication during shift turnover between personnel responsible for the work which adjusted the limit switches. This resulted in the close limit switch being left without the final adjustment being completed.

Requalification training will be provided to the maintenance electricians and supervisors on written communications to be used on work completion forms, requirements for review of completed work packages and paperwork sign-off in procedures.

FACILITY NAME (1)	1	DOC	KET	NUN	ABER	(2)							ER N	UMB.	ER (8	1				P	AGE	3)	
											YE	AR	SEC N	UENT UNRIE	RAL R		R	EV O					
Callaway Plant Unit 1		0	5	0	0	0	4	8	1:	3	9	2	0	1	2		0	0	0	2	OF	0	6

TEXT 01 more space is required, use additional NRC Form 366A's)(17)

## BASIS FOR A VOLUNTARY REPORT:

On 10/30/92, Essential Service Water (ESW) Ultimate Heat Sink (UHS) cooling tower train 'B' bypass valve EF-HV-0066 was found to be approximately 25% open when the remote valve position indications indicated closed. This condition had existed since 4/12/92 following valve actuator maintenance which had adjusted limit switches during Refuel 5. The 'A' ESW train has been inoperable for maintenance and surveillance at various times since 4/12/92. The 'B' ESW train was initially determined inoperable on 10/30/92 and a phone notification was made to the Nuclear Regulatory Center Operations Center per 10CFR50.72(b)(2)(iii) on 10/30/92 at 1515 CST to report the inoperability of both trains of ESW. Subsequently, the 'B' ESW train was determined to be operable in its as found positior, based on a utility engineering evaluation supported by an analysis completed by the architect engineer. The 4-hour phone call made 10/30/92 that reported the inoperability of both trains of Essential Service Water (ESW) at various times from 4/12/92 to 10/30/92 was retracted 11/25/92 at 1625 CST.

### CONDITION AT TIME OF EVENT:

At Time of Event: Mode 6 - Refueling

At Time of Event Discovery: Mode 1 - Power Operations; 100% Reactor Power

## DESCRIPTION OF EVENT:

On 4/09/92, planned maintenance was completed on butterfly valve EF-HV-0066, ESW UHS cooling tower train 'B' bypass valve<sup>(1)</sup>. Following completion of the work, a utility maintenance electrician adjusted the valve actuator limit switches by procedure. Attachment 4, Note 3, to procedure MTM-ZZ-QA006, Limitorque Actuator Limit Switch and Torque Switch Adjustment, instructed the electrician to set the Intermediate 2 rotor such that the valve disc would coast into the seat after the close limit switch contacts opened. The Intermediate 2 limit switch rotor close contacts de-energize the valve actuator when the valve is closing. The electrician set the close limit switch at 20 hand wheel turns (corresponding to approximately 25% open) from the valve fully closed position and recorded that this was the position set on the Work Request (WR) completion form. The second shift electricians read this to mean that the limit switch was adjusted to the appropriate position, and went on to place the remaining limits.

FACILITY NAME (1)	00	CKET	NUN	HBER	1151				1	*****	erc domen	LER N	UMB	ER 18	3)		enclose		P	AGEI	3)	
									Y	EAR		SEC	UMBR	RAL R		RIN	6V 0.					
Callaway Plant Unit 1	0	5	0	0	0	4	8	100	9	2		0	1	2		0	0	0	3	OF	0	6

TEXT iff more space is required, use additional NRC Form 366A s1(17)

On 4/13/92, a valve retest was performed per procedure MTE-ZZ-QA001, Baseline MOVATS Testing of Limitorque Motor Operated Valves. The MOVATS test coordinator ensured the limitorque minimum and maximum thrusts requirements and the maximum stroke time were met. The valve stroked in 19.4 seconds, the maximum allowable stroke time is 35 seconds. The procedure did not require verification of full valve closure or comparison of the stroke time io previous data. Since the readings were satisfactory, the valve passed the MOVATS procedure acceptance criteria.

Valve EF-HV-0066 was surveilled for operability three times after 4/13/92. Procedure OSP-EF-V001B, ESW Train 'B' Valve Operability, was performed on EF-HV-0066 on 4/14/92, 7/29/92 and 10/21/92. This surveillance is satisfied by non-licensed operators locally observing the movement of a position indicator<sup>(2)</sup> attached to the valve stem as the valve is cycled by remote control from the Main Control Board (MCB)<sup>(3)</sup>. This was compared against remote valve position indicator are approximate indicators of the valve disc position and the failure of the indicator associated with EF-HV-0066 to fully rotate through 90 degrees of movement was not noted. There was no positive indication of closed position to compare the indicator.

On 10/30/92, a utility systems engineer performing MOVATS trending evaluations for future maintenance identified inconsistencies in test results that led him to question the settings of the limit switch. Subsequent inspection the same day determined the close limit switch was set to stop valve motion when the valve was approximately 25% from the full closed position. The valve position was corrected immediately by manually closing the valve.

#### ROOT CAUSE:

The root cause of this event is inadequate communication during shift turnover between the personnel responsible for the work. This resulted in the close limit switch being left without the final adjustment being completed.

#### CONTRIBUTING FACTORS:

 Procedure MTM-ZZ-QA006 did not give exact guidance concerning the setting of the close limit switch for butterfly valves. The electrician set the close limit switch 20 hand wheel turns from the closed seat with the incorrect understanding that subsequent personnel working on the valve would complete the adjustment. Electricians on the next shift thought that this portion had been completed and did not question the settings, but proceeded to complete the remaining limit switch settings.

FACILITY NAME (1)	DO	OKET	NU	ABER	.121				1		-	ER N	UMB	ER 16	5)				P)	AGE I	3)	
									YE	AB		SEC	UNINT	TAL. Pl		RE N	V D.					
Callaway Plant Unit 1	0	5	0	0	0	4	в	3	9	2		0	1	2		0	0	0	4	ÖF	0	6

TEXT III more space is required, use additional NRC Form 386A #/(17)

- The second shift electrician recording information in the procedure copied information from the completion form and did not verify that the as-left condition assured the valve closed completely. These steps in the procedure were not signed off and this was not identified during package review by the supervisor.
- Although not directly related to the error which was made in setting the close limit switch, there
  was general misunderstanding of the capability of the subsequent MOVATS testing to effectively
  verify butterfly valve seat positioning.
- Procedure MTM-ZZ-QA006 does not require the valve to be energized and stroked to verify limit switch settings.
- Non-licensed operators failed to identify EF-HV-0066 was not going fully closed via local observation during valve retests.

### CORRECTIVE ACTIONS:

- Procedure MTM-ZZ-QA006 will be revised by 3/1/93 to give better guidance on how to set up a limitorque operator on a butterfly valve, including: a starting point for the close limit switch and energizing and stroking the valve to verify settings. In the interim period, supplemental oversight will be provided for similar valve maintenance activities.
- Requalification training will be provided to the maintenance electricians and supervisors by 9/1/93
  on the following:
  - a. MOVATS testing as used on butterfly valves.
  - b. Revised MTM-ZZ-QA006 procedure.
  - c. Training on written communications to be used on work completion forms.
  - d. Procedural requirements for paperwork sign-offs in procedures.
  - e. Requirements for review of completed work packages for supervisors.
- MOVATS Test Coordinators will be provided with data by 1/1/93 from the previous baseline MOVATS testing to assist in evaluating if any abnormalities are evident.
- MTE-ZZ-QA001 will be revised by 1/1/93 to provide clarification in evaluating abnormalities such as comparing the switch trace with the previous baseline data for stroke times and bypass switch settings.

FACILITY NAME (1)	DO	CKET	NUR	ABER	(2)		-	-			1	ER N	UMB	ER (6	12				P.	AGE	31	
									YE	AR.		SEC	URNI	hal H		RIN	EV O.					
Callaway Plant Unit 1	0	5	0	0	0	4	3	3	9	2		0	1	2	-	0	0	0	5	OF	0	6

TEXT SI more space is required, use additional NRC Form 368A's)(17)

- 5. Trend reviews of MOVATS testing will be completed by Engineering within 60 days.
- 6. Non-licensed operators will be trained by 4/1/93 on what to specifically look for when verifying local indications for Section XI tests. If indications are not present or accurate, personnel will be instructed to initiate the appropriate corrective actions.
- 7. Proper limit switch adjustment was verified and EF-HV-0065, the corresponding valve on 'A' train of ESW, was verified by direct observation on 11/2/92 to be stroking fully closed. Other butterfly valves other than EF-HV-066 and EF-HV-0065 which have a safety-related function to close have been identified. Positive means exist through conduct of post maintenance retests, surveillance tests, or system performance monitoring to verify other valves are fully closed.

#### SAFETY SIGNIFICANCE:

The 'B' train ESW system was available for heat removal. Although valve EF-HV-0066 was 25% from its full closed position, the 'B' ESW train would have performed its intended function. All of the 'B' ESW flow would have rejected heat to the UHS pond as designed until a high return temperature closed the valve to force the ESW flow to the cooling tower for increased heat rejection. Approximately 50% of the 'B' ESW flow would have been directed to the cooling tower, while the remaining 50% would have continued to reject the heat to the UHS pond. This was verified by field testing with the valve positioned in the as found position. T' e 'A' train ESW system was available for heat removal with the exception of planned maintenance outages.

Based on an engineering review, valve EF-HV-0066 would have allowed sufficient flow of water to the UHS cooling tower such that adequate heat rejection would have occurred. The engineering review which was performed included adequate conservatism to ensure that equipment relying on this cooling would have performed its safety function during a design basis accident. Therefore, there was no threat to the public health or safety.

PREVICUS OCCURRENCES:

None.

FACILITY NAME (1)	DO	CKE	T NU	ABER	(2)		want of the				SP ADD DO NO	ER N	UMB	ER 18	0		CarGenties		p,	AGE	2)	1011.05.004
									YE	AR		SEC	UMBE	RAL FR		RI	D.					
Callaway Plant Unit 1	0	5	0	0	0	4	8	3	9	2		0	1	2		0	0	0	6	OF	0	6

TEXT (If more space is required, use additional NPC Form 366A's)(17)

## FOOTNOTES:

\*

ж.

The system and component codes listed below are from the IEEE Standard 805-1985 and 803A-1984.

- (1) System BI, Component FLV
- (2) System BI, Component ZI
- (3) System BI, Component IL