Westinghouse Electric Corporation
Water Reactor Divisions

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May 30, 1985

NS-NRC-85-3038

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20055

50-498/499

Attention: P. Kadambi

Transmittal of Westinghouse Documents Requested
by the USNRC at the South Texas Mechanical Engineering Branch
Audit on May 17, 1985

Enclosed is one copy each of the subject documents which are listed below. These documents contain information which is proprietary to Westinghouse. Accordingly, we request that this information be withheld from public disclosure. By copy of this letter, one copy each of these documents is being transmitted to NRC consultants E. C. Rodabaugh Associates, Inc., Hilliard, Ohio, and one copy each to Sam Moore, Y-12 Plant, Oak Ridge, Tennessee, at the request of Mr. Victor Nerses, NRC, LB-3, for the purpose of expediting review.

Document Number	Document Type	Title
G-952342 Rev. 2 with interim revisions 1 & 2	Equipment Specification	General Specification Reactor Coolant Pump Model 100 60 Hertz & Model 100 50 Hertz (Proprietary)
952721 Rev. 4	Equipment Specification	South Texas Reactor Coolant Pump (Proprietary)
WEMD E.M. 5351	Engineering Memorandum	South Texas Project Unit No. 1 Model 100 Reactor Coolant Pump Pressure Boundary Summary Report (Proprietary)
WEMD E.M. 4932, Rev. 1, Volume 1 & Volume 2	Engineering Memorandum	Model 100 Reactor Coolant Pump - Pressure Boundary Stress Report (Proprietary)
WEMD E.M. 5003, Rev. 1	Engineering Memorandum	Model 100 Reactor Coolant Pump - Casing Nozzle Design Loads (Proprietary)

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Document Number	Document Type	Title
G-952850, Rev. 0 with interim revision 1	Equipment Specification	Motor Operated Gate Valves - ASME Boiler and Pressure Vessel Code Section III - Class 1, 2, & 3 (Proprietary)
952874, Rev. 4	Design Specification	South Texas Project Units Number 1 and 2 Motor Operated Gate Valves - ASME Boiler and Pressure Vessel Code Section III (Proprietary)
WEMD E.M. 5158 Volumes 1 & II	Engineering Memorandum	Stress Report for Westinghouse Class I Nuclear Valves 6 inch and larger Gate Valves for Westinghouse PWRSD (Proprietary)
8373D81, Rev. 3	Engineering Drawing	Motor Op Gate Valve Mod. 08000GM84FEB0D0 8-316 ASME Cl 1 GPO Assy. (Proprietary)
953385, Rev. 0	Equipment Specification	Piping Design Specification ASME III Code Class 1, for Houston Lighting and Power Co. - South Texas Project Nuclear Power Plants Units 1 and 2 (Proprietary)
953385, Rev. 1	Design Specification	Piping Design Specification ASME III Code Class 1, for Houston Lighting & Power Co. - South Texas Project Nuclear Power Plant Units 1 and 2 (Proprietary)
WCAP-9135	Westinghouse Report	Structural Analysis of Reactor Coolant Loop for the South Texas Project Units 1 & 2 -Volume 1 - Analysis of the Reactor Coolant Loop Piping (Proprietary)
G-952744, Rev. 0	Equipment Specification	Reactor Coolant System Equipment Supports Design Specification, ASME III Code Class 1 (for Steam Generator, Reactor Coolant Pump, Reactor Pressure Vessel, and Pressurizer) (Proprietary)

Document Number	Document Type	Title
953533, Rev. 0	Equipment Specification	South Texas Project Units 1 & 2 Reactor Coolant System Equipment Supports Design Specification, ASME III Code Class 1 (for Steam Generator, Reactor Coolant Pump, Reactor Pressure Vessel, and Pressurizer) (Proprietary)
G-952628, Rev. 1	Equipment Specification	Fabrication Requirements for the Reactor Coolant System Component Supports (Proprietary)
SSDC 1.3, Rev. 2	Systems Standard Design Criteria	Systems Standard Design Criteria - Nuclear Steam Supply System - Design Transients (Proprietary)
SSDC 1.3, App. A Rev. 0	Systems Standard Design Criteria	Fluid Systems Design Transients 3XL and 4XL (South Texas) Plants (Proprietary)
CENC-1332	Analytical Report	Addendum 1 to Analytical Report for South Texas Project No. 1 - Houston Lighting and Power Co. (Proprietary)
WCAP-8447	Westinghouse Report	Roto-Lok Closure System Development (Proprietary)

Two of the requested documents, RPT-MED-PCE-577, Rev. 1, and WCAP-10197, are not included because they do not apply to the South Texas Projects Units 1 and 2. These two documents are available for review at the Westinghouse Nuclear Center.

There has been no generic work done to date to address out-of-roundness of piping products. The Code equations have been reviewed for a sample of components and the additional stress associated with out-of-roundness was found to be small. The formal consideration of these additional stresses will be included in the documentation associated with the final stress report.

The finite element analysis performed on the reactor coolant loop elbows to determine flexibility factors and stress indices is considered proprietary and will not be released in its entirety. If there are specific questions on this

information or if you would accept a general discussion of what analysis was performed, please communicate your needs to Westinghouse and we will attempt to satisfy you.

In order not to delay this submittal of information requested by the Commission, we will comply with the requirements of 10 CFR 2.790 to provide proprietary and non-proprietary versions together with an affidavit as soon as the proprietary information contained in the submittal has been specifically identified and the proprietary and non-proprietary versions have been prepared. We will submit the total required number of copies of the proprietary and non-proprietary versions of the information and the required affidavit at that time.

In the meantime, we have provided sufficient copies for you to initiate your review. E. Shomaker, Esq., of the NRC Office of the Executive Legal Director, has advised Westinghouse that he concurs with this procedure.

We expect to be able to fully comply with the requirements for the proprietary and non-proprietary versions of the information and an accompanying affidavit within four weeks.

Very truly yours,

WESTINGHOUSE ELECTRIC CORPORATION

E. P. Rahe, Jr., Manager
Nuclear Safety Department

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Attachments

cc: E. C. Shomaker, Esq.
Office of the Executive Legal Director, NRC