

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Indian Point, Unit 3										DOCKET NUMBER (2) 0 5 0 0 0 2 8 6 1 OF 0 1										PAGE (3) 1 OF 0 1	
TITLE (4) Unit Trip																					
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)											
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES					DOCKET NUMBER(S)							
0 3	1 9	8 5	8 5	0 0 3	0 0	0 4	1 5	8 5						0 5 0 0 0							
THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)																					
OPERATING MODE (9)		N		20.402(b)				20.405(c)				<input checked="" type="checkbox"/> 50.73(a)(2)(iv)				73.71(b)					
POWER LEVEL (10)		0 8 5		20.405(a)(1)(i)				50.36(c)(1)				50.73(a)(2)(v)				73.71(c)					
				20.405(a)(1)(ii)				50.36(c)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)					
				20.405(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)									
				20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)									
				20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(ix)									
LICENSEE CONTACT FOR THIS LER (12)																					
NAME John J. Anderson										TELEPHONE NUMBER 9 1 4 7 3 9 - 1 8 2 0 1 0											
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																					
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPD		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPD											
X	JIC	5 2 1	W 1 2 1 0	N																	
SUPPLEMENTAL REPORT EXPECTED (14)																					
YES (If y complete EXPECTED SUBMISSION DATE)												NO									
												EXPECTED SUBMISSION DATE (15)									
												MONTH DAY YEAR									

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At 1314 hours on March 19, 1985 with reactor at 85 percent power, reactor trip breaker RTB (Westinghouse, Model DB-50) opened, initiating a unit trip. All applicable plant parameters were at normal levels at the time. Extensive testing was performed on the coils and relays for both reactor trip breakers (RTA and RTB). No irregularities were found. After testing was completed, preventive maintenance procedure 3-PM-A-ES-13 was performed with satisfactory results. The appropriate sections of surveillance tests 3PT-M13A and 3PT-M13B were then performed to verify the operability of both breakers. All equipment associated with the reactor trip operated correctly. It has been postulated that the manual trip lever for breaker RTB may have been inadvertently actuated by personnel working in the area. This could not be substantiated. No similar events have been reported in an LER to date.

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PDR ADCK 05000286  
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Indian Point 3  
Nuclear Power Plant  
P.O. Box 215  
Buchanan, New York 10511  
914 739.8200



April 15, 1985  
IP-FWG-864

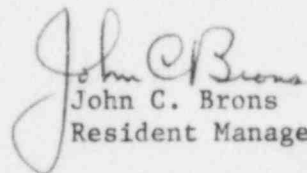
Docket No. 50-286  
License No. DPR-64

Document Control Desk  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Dear Sir:

The attached Licensee Event Report LER 85-003-00 is hereby submitted in accordance with the requirements of 10CFR50.73. This event is of the type defined in Paragraph 50.73(a)(2)(iv).

Very truly yours,

  
John C. Brons  
Resident Manager

FWG/bam  
Attachment

cc: Dr. Thomas Murley  
Regional Administrator  
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U. S. Nuclear Regulatory Commission  
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King of Prussia, Pennsylvania 19406

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