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The Northeast Utilities System

JUL 24 1998

Docket No. 50-443 NYN-96049

United States Nuclear Regulatory Commission Attn.: Document Control Desk Washington, D.C. 20555

Seabrook Station
Licensee Event Report (LER) 96-04-00
Emergency Feedwater System Valve Closure

Enclosed, please find Licensee Event Report (LER) No. 96-004-00 for Seabrook Station. This submittal documents an event which occurred on June 27, 1996. This event is being reported pursuant to 10CFR50.73(a)(2)(ii).

Should you require further information regarding this matter, please contact Mr. Anthony M. Callendrello, Licensing Manager, at (603) 474-9521, extension 2751.

Very truly yours,

NORTH ATLANTIC ENERGY SERVICE CORP.

William A. DiProfio Station Director

cc:

T. T. Martin, Regional Administrator

A. W. De Agazio, NRC Project Manager, Seabrook Station J. B. Macdonald, Senior Resident Inspector, Seabrook Station

INPO Records Center 700 Galleria Parkway Atlanta, GA 30339

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NRC FORM (4-95)	A 366	U.S. NUCLEAR REGULATORY COMMISSION  LICENSEE EVENT REPORT (LER)								EXPIRES 04/30/98  ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS MANDATOR: INFORMATION COLLECTION REQUEST. 50.0 HRS. REPORTED LESSONS LEARNED ARE INCORPORATED INTO THE LICENSING PROCESS AND FEE BACK. TO INDUSTRY. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH IT 6 F33]. U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (13/50.0104).									
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On June, 27, 1996, North Atlantic Energy Services Corporation determined that the alignment of the Startup Feedwater Pump (SUFP) to the Steam Generators (SG) via the Emergency Feedwater (EFW) flow control valves was an unanalyzed condition that significantly compromised safety. A one hour non-emergency report was made pursuant to 10CFR 50.72(b)(1)(ii).

The Seabrook Station Auxiliary Feedwater System is comprised of 2 - 100% capacity Emergency Feedwater (EFW) pumps and one 100% capacity Startup Feedwater Pump (SUFP). The SUFP is normally used to supply feedwater to the SG's during startup and shutdown evolutions. However, the SUFP has the capability of being aligned to the EFW System and functioning as a third EFW pump in the unlikely event that the normal EFW pumps are unavailable.

Station Operating Procedures were developed to align the SUFP to supply feedwater to the SGs through the EFW flow control valves. This procedure section was intended to be used when the normal feedpath was unavailable due to maintenance. This alignment was not adequately analyzed and the consequences of a postulated loss of all AC power may have been worsened by the alignment.

The root cause of this event was determined to be an inadequate 10CFR50.59 safety evaluation and Technical Clarification in that the evaluations were performed in a qualitative manner whereas a rigorous safety evaluation was warrented.

NRC FORM 366A (4-95)

U.S. NUCLEAR REGULATORY COMMISSION

## LICENSEE EVENT REPORT (LER)

TEXT CONTINUATION

FACILITY NAME (1)	DOCKET NUMBER (2)		PAGE (3)				
Seabrook Station	05000443	YEAR	SEQUENTIAL NUMBER			REVISION NUMBER	2 of 3
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TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

### I. Description of Event

On June, 27, 1996, North Atlantic Energy Services Corporation determined that the alignment of the Startup Feedwater Pump (SUFP) to the Steam Generators (SG) via the Emergency Feedwater (EFW) flow control valves was an unanalyzed condition that significantly compromised safety. A one hour non-emergency report was made pursuant to 10CFR 50.72(b)(1)(ii).

The EFW System is comprised of two full-sized pumps (one motor and one turbine driven), whose water source is the Condensate Storage Tank. Both pumps feed a common discharge header, which in turn supplies the four emergency feed lines. Each emergency feed line has two normally open motor operated valves in series. Additional pumping capacity is provided by the SUFP in the feedwater system. The SUFP connects to the EFW system through two normally closed motor operated valves in series.

Seabrook Station developed operating procedures which aligned the SUFP to the EFW discharge header. These procedures were intended to be used during infrequent maintenance activities which required isolation of the normal feed path. The procedure changes were initially implemented without a safety evaluation. A subsequent safety evaluation supported the changes and determined that an unreviewed safety question did not exist. The 10CFR50.59 safety evaluation relied on qualitative assumptions regarding operator response if a valid EFW actuation were to occur while operating in the configuration in question. The evaluation did not identify statements in the UFSAR that appear to prohibit operation in this configuration. These same qualitative assumptions were relied on in a Technical Clarification which also supported the configuration in question. The qualitative assumption that operators would restore EFW flow control valves and EFW pump recirculation valves to their required positions in a timely manner upon receiving a valid EFW actuation appears to be a reasonable assumption yet a rigorous evaluation of these assumptions has not been performed to fully ensure that the design/licensing basis of the EFW System has not been undermined.

Under the assumption that a station blackout occurs during the time that the EFW flow control valves are throttled/closed, there is adequate procedural guidance available to reasonably assure that an adequate secondary heat sink is maintained provided the valves are initially throttled to a position corresponding to the required EFW flow. The procedure for aligning the SUFP to the EFW header initially closes the EFW flow control valves. If a loss of all AC power was postulated to occur while the flow control valves were closed, it is possible that operators would not be able to locally open the valves and reestablish flow before the consequences of the accident were significantly worsened.

In 1991, North Atlantic evaluated the alignment of the Condensate Pumps to the EFW header and inappropriatly determined that the event was not reportable. This configuration also throttles the EFW flow control valves. The plant was placed in this configuration to allow maintenance to be performed on the check valves in the normal feedwater path to each steam generator while the plant remained in Mode 3. The plant was in this alignment from April 2, 1991 to April 5, 1991.

### II. Cause of Event

The root cause of this event was determined to be an inadequate 10CFR50.59 safety evaluation and Technical Clarification in that the evaluations were performed in a qualitative manner whereas a rigorous quantitative safety evaluation was warranted.

NRC FORM 366A

U.S. NUCLEAR REGULATORY COMMISSION

# LICENSEE EVENT REPORT (LER)

TEXT CONTINUATION

FACILITY NAME (1)	DOCKET NUMBER (2)		PAGE (3)				
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#### III. Analysis of Event

Under the conditions described above, the alignment of the SUFP pump to the to the EFW flow control valves constitutes an unanalyzed condition that significantly compromised safety. It is important to note that this conclusion is predicated on the highly improbable assumption that the EFW flow control valves will be closed concurrent with a loss of all AC power event.

### IV. Corrective Action

North Atlantic has deleted the procedure section which aligned the SUFP to the EFW pump discharge header. Other procedures which could potentially place the plant into a similar configuration will be reviewed and revised or cancelled as appropriate.

North Atlantic is in the process reviewing recently identified inconsistencies within the UFSAR and other Current Licensing Basis Documents. Corrective actions under evaluation include enhanced 10CFR50.59 safety evaluation training and guidance regarding when a safety evaluation should be performed. North Atlantic will specify the corrective action in a follow up report to be submitted by September 30, 1996.

#### V. Additional Information

Similar Events

None

Manufacturer Data

Not applicable

#### Plant Conditions

At the time of this event, the plant was operating in Mode 1 at 100% power with the Reactor Coolant System temperature of 587.5 degrees Fahrenheit and pressure of 2235 psig.