

Duke Power Company
Catawba Nuclear Station
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DUKE POWER

October 21, 1992

RE: Catawba Nuclear Station
Selected Licensee Commitments
Effective 10/21/92

Attached are revisions to the Catawba Nuclear Station Selected Licensee Commitments. Please revise your copy as follows:

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Any questions should be directed to the undersigned at
803-831-3237.

Kay E. Nicholson
Kay E. Nicholson
Regulatory Compliance

Attachments

.SLC.LTR

070021
9211020145 921021
PDR AJOCK 05000413
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CATAWBA NUCLEAR STATION
FINAL SAFETY ANALYSIS REPORT
SELECTED LICENSEE COMMITMENTS
CHAPTER 16.7

INSTRUMENTATION

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16.7-1 ATWS Mitigation System Actuation Circuitry (AMSAC)

16.7 INSTRUMENTATION

16.7 ATWS MITIGATION SYSTEM ACTUATION CIRCUITRY (AMSAC)

COMMITMENT:

The AMSAC System shall be OPERABLE.

APPLICABILITY:

MODE 1, above 40% of RATED THERMAL POWER (based on Turbine Impulse Pressure).

REMEDIAL ACTION:

With the AMSAC System inoperable, restore it to OPERABLE status within 7 days or submit a Special Report to the Nuclear Regulatory Commission within the following 30 days. This report shall outline the cause of the malfunction and the plans for restoring the system to OPERABLE status.

TESTING REQUIREMENTS:

Perform a CHANNEL CALIBRATION on the AMSAC System instruments at least once per 18 months.

REFERENCES:

- 1) 10 CFR 50.62, Requirements for Reduction of Risk From Anticipated Transients Without Scram (ATWS) Events for Light-Water Cooled Nuclear Power Plants.
- 2) Generic Letter 85-06, "Quality Assurance Guidance for ATWS Equipment That Is Not Safety-Related."
- 3) Information Notice 92-06, "Reliability of ATWS Mitigation System and Other NRC Required Equipment Not Controlled by Plant Technical Specifications."

BASES:

Per 10 CFR 50.62, "Each pressurized water reactor must have equipment from sensor output to final actuation device, that is diverse from the reactor trip system, to automatically initiate the auxiliary feedwater system and initiate a turbine trip under conditions indicative of an ATWS. This equipment must be designed to perform its function in a reliable manner and be independent from the existing reactor trip system." When this rule was issued, the NRC did not require licensees to address the OPERABILITY of this equipment in the plant Technical Specifications nor require that this equipment be designated as safety-related.

On January 15, 1992, the NRC issued Information Notice 92-06 which discussed the reliability of AMSAC and other equipment not controlled by plant Technical Specifications. This notice described two separate incidents where violations were cited because the licensees failed to adequately maintain the reliability of their AMSAC systems. The NRC is concerned that licensees may not place an appropriate level of priority on resolving problems with the AMSAC System because it is not a safety-related system and because the plant's Technical Specifications do not govern its operability. The NRC considers the failure of licensees to adequately ensure the reliable operation of AMSAC equipment to be a significant regulatory concern.

This Selected Licensee Commitment was developed to ensure that appropriate attention is given to maintaining the AMSAC System in a reliable condition and that prompt action will be taken to repair and restore any AMSAC equipment that is discovered in a condition where it is incapable of performing its intended function.