AVERAGE DAILY UNIT POWER LEVEL

Docket No. 50-272
Unit Name Salem # 1
Date April 10,1985
Telephone 609-935-6000
Extension 4455

Completed by J. P. Ronafalvy

Month March 1985

Day	Average Daily Power Level (MWe-NET)	Day	Average Daily Power Level (MWe-NET)
1	1098	17	1115
2	1114	18	1105
3	1104	19	1102
4	1115	20	1102
5	1101	21	1100
6	1099	22	1101
7	976	23	1106
8	1084	24	1105
9	1102	25	1114
10	1102	26	1087
11	1092	27	1107
12	1106	28	1103
13	1103	29	1111
14	1095	30	1077
15	1092	31	1102
16	1111		

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1/1

OPERATING DATA REPORT

Docket No. 50-272
Date April 10, 1985
Telephone 935-6000
Extension 4455

Completed by J. P. Ronafalvy

Operating Status

1.	Unit Name	Salem No. 1	Notes
2.	Reporting Period	March 1985	
3.	Licensed Thermal Power (MWt)	3338	
4.	Nameplate Rating (Gross MWe)	1170	
5.	Design Electrical Rating (Net MWe)	1090	
6.	Maximum Dependable Capacity (Gross MWe)	1124	
7.	Maximum Dependable Capacity (Net MWe)	1079	

8. If Changes Occur in Capacity Ratings (items 3 through 7) since Last Report, Give Reason N/A

9. Power Level to Which Restricted, if any (Net MWe) N/A

10. Reasons for Restrictions, if any N/A

	This Month	Year to Date	Cumulative
11. Hours in Reporting Period	744	2160	67969
12. No. of Hrs. Reactor was Critical	744	2142.6	37966.1
13. Reactor Reserve Shutdown Hrs.	0	0	3088.4
14. Hours Generator On-Line	744	2139.7	36298.3
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	2480827	7082904	109850901
17. Gross Elec. Energy Generated (MWH)	848600	2425140	36341190
18. Net Elec. Energy Generated (MWH)	816628	2331205	34429187
19. Unit Service Factor	100	99.1	53.4
20. Unit Availability Factor	100	99.1	53.4
21. Unit Capacity Factor (using MDC Net)	101.7	100.0	46.9
22. Unit Capacity Factor (using DER Net)	100.7	99.0	46.5
23. Unit Forced Outage Rate	0	0.9	31.9

24. Shutdowns scheduled over next 6 months (type, date and duration of each)

N/A

25. If shutdown at end of Report Period, Estimated Date of Startup:
N/A

26. Units in Test Status (Prior to Commercial Operation):

	Forecast	Achieved
Initial Criticality	9/30/76	12/11/76
Initial Electricity	11/1/76	12/25/76
Commercial Operation	12/20/76	6/30/77

UNIT SHUTDOWN AND POWER REDUCTIONS
REPORT MONTH March 1985

Completed by J.P. Ronafalvy

Docket No. 50-272
Unit Name Salem No.1
Date April 10,1985
Telephone 609-935-6000
Extension 4455

No.	Date	Type 1	Duration Hours	Reason 2	Method of Shutting Down Reactor	License Event Report	System Code 4	Component Code 5	Cause and Corrective Action to Prevent Recurrence
85-182	3-07	F	8.5	A	5	-	HB	VALVEX	Intercept Valves Turbine

1
F: Forced
S: Scheduled

2 Reason
A-Equipment Failure-explain
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & Licensing Exam
F-Administrative
G-Operational Error-explain
H-Other-explain

3 Method
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuation of Previous Outage
5-Load Reduction
9-Other

4 Exhibit G Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG 0161)

5 Exhibit 1 Salem as Source

MAJOR PLANT MODIFICATIONS
REPORT MONTH MARCH 1985

DOCKET NO.: 50-272
UNIT NAME: Salem 1
DATE: April 10, 1985
COMPLETED BY: J. Ronafalvy
TELEPHONE: 609/339-4455

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
1EC-1304	Chemical & Volume	Buttwelding of valves (associated with the test, vent and drain connections along the centrifugal and reciprocating charging pump suction and discharge lines and CVC letdown Lines) to the header making each of them a "one piece connection"
1EC-1588	Cable Trays	Retag, repull various cables throughout the plant to match changes made to wiring diagrams prior to implementation of the DCR system.
1EC-1676A	Reactor Coolant	Replace reactor coolant narrow range RTD's with environmentally qualified elements. Also relocate solenoid valves associated with MS7 and MS18 valves.
1EC-1698A	Circulating Water-Traveling Screens	Jnstall a 316 stainless steel traveling screen in place of 138 screen. Install one set of 316 stainless steel baskets in 11A traveling screen. Install flexible line from inlet screen wash piping to screen wash spray hdeader piping.
1EC-1775	Circulating Water	Replace the inlet expansion joints of Unit 1 Main Condensers, additionally replace the remaining outlet expansion joints not changed during the main condenser retubing in 1980.

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
1EC-1816	Safeguards Equipment Controller R130	Install new Lambda power supplies and new Potter Brumfield relays in the Unit 1 SEC's.
1EC-1830	Generator Protective Relay	Upgrade generator stator ground fault relay protection.
1EX-1836	Reactor Coolant	Revise setpoints of #12 RCP locked rotor protection relaying.
1EC-1874A	Service Water	Revise 8" Service Water emergency supply line to Aux Feedwater from #12 Service Water nuclear header. Revise 4" SW supply line to room and oil coolers from #12 Service Water nuclear header. Line or replace stainless steel service water supply and return piping to #12 CC Hx. Add break flanges in service water supply and return piping to #12 CC Hx.
1EC-1950	Containment Cooling Fan Coolers	Machine stator core, rotor and roll file to remove eccentricity between stator and core and obtain uniform air-gap. Also increase air gap from 39 mils to 44 mils. Drill ports (3/4") which can be plugged, on end bell - 4 each on each end.
1EC-1988	Steam Generator Feed Pump and Turbine Lube and Control Oil	Change the SGFP No. 11 and 12 thrust bearing wear alarm and trip setpoints from 8 and 12 psi to 13 and 17 psi respectively.
1ET-1996	Chilled/Service Water	Test application of Garlock "Turbo-Star" cartridge type mechanical seal in chiller condenser recirc pumps No. 11, 12, 13.

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
1SC-0605	Service Water Pump Motors (#15)	Vent the oil level columns in the bearing oil lubrication system on No. 15 Service Water Pump Motor.
1SC-0892	Containment Personnel Hatch	Install a mechanical stop valve at the hatch flowmeter-air supply line. Separate valve to be installed on El. 100' and El. 130' hatches.
1SC-1306	Security (Barriers)	Make additions and modifications to provide security barriers on three air vents in the 122' El. hallway between the Aux Building and the "A" Building.

MAJOR PLANT MODIFICATIONS
REPORT MONTH MARCH 1985

DOCKET NO.: 50-272
UNIT NAME: Salem 1
DATE: April 10, 1985
COMPLETED BY: J. Ronafalvy
TELEPHONE: 609/339-4455

*DCR NO.	SAFETY EVALUATION 10 CFR 50.59
1EC-1304	This change addresses the intent to increase conservatism and thereby reduce the possibility of a future occurrence of cracks/breaks at the test, vent, and drain connections. In addition, the section modulus of the connection will be significantly increased which will improve resistance to heretofore undetected shock load. No unreviewed safety or environmental questions are involved.
1EC-1588	The work to be done with this DCR will not change the actual design of any system nor will it affect any safety related system. No unreviewed safety or environmental questions are involved.
1EC-1676A	This change involves a substitution of existing equipment. The system response time will increase and the over temperature delta T (Table 3.3.2 of the Tech. Specs.) will increase but by an amount less than that allowed in Table 15.1-3 of the FSAR accident analysis. No unreviewed safety or environmental questions are involved.
1EC-1698A	This change involves the upgrading of the traveling screen material. This system does not affect the presently performed safety analysis nor does it create any new hazards. No unreviewed safety or environmental questions are involved.
1EC-1775	The work to be done with this DCR will not change the actual design of any system nor will it affect any safety related system. No unreviewed safety or environmental questions are involved.
1EC-1816	This change replaces components with components manufactured by a different vendor. All replacement components have been qualified to equal to or greater requirements than the original equipment. This includes seismic qualification to IEEE 344. All failure modes are identified to the originally installed equipment. No unreviewed safety or environmental questions are involved.

*DCR - Design Change Request

*DCR NO.	SAFETY EVALUATION 10 CFR 50.59
1EC-1830	This change increases protection of the Main Generator stator from ground faults. The change is non-safety related and does not add any generator trips which are not already possible through various electrical faults. No unreviewed safety or environmental questions are involved.
1EX-1836	This change verifies the #12 RCP locked motor protection relay set points to determine if the motor is acceptable to remain in service or must be removed for further inspection. No unreviewed safety or environmental questions are involved.
1EC-1874A	This change replaces the Service Water unlined stainless steel supply and return piping with polyethylene lined carbon steel. The new piping will prevent the buildup of marine growth on the pipe wall and will prevent pipe corrosion. The new polyethylene piping will stand up to the Service Water conditions. It will not affect the effluent to the river and will prevent the release of Service Water to the atmosphere. Also, the 8" emergency supply to the Auxiliary Feedwater System and the 4" supply to the Room Coolers are being revised to allow the line to be isolated thereby eliminating the build-up of marine growth and silt. It will also allow the supply line to the Room Coolers to operate more efficiently. The addition of break flanges will allow for future inspections to verify the integrity of the system. No unreviewed safety or environmental questions are involved.
1EC-1950	These changes result in a uniform air gap in the electrical motor and provides a means to verify the uniformity. These features only add to the reliability and availability of the motor. No unreviewed safety or environmental questions are involved.
1EC-1988	The intended function of the system remains unaffected. The change is in conformance with the manufacturer's recommendation. This is a non-safety related system. No unreviewed safety or environmental questions are involved.
1ET-1996	The test applicatins of a "Garlock Turbostar" cartridge type mechanical seal in the Chiller Condenser Recirc. Pumps do not affect any presently performed safety analysis nor does it create any new hazards. No unreviewed safety or environmental questions are involved.

*DCR - Design Change Request

*DCR NO.	SAFETY EVALUATION 10 CFR 50.59
1SC-0605	The modification of the oil sight gauge vent will improve the accuracy of the oil level indication. No unreviewed safety or environmental questions are involved.
1SC-0892	The controls changes do not affect the operation of installed equipment and will not deduct from the margin of safety of presently installed equipment. This change does not affect the safe shutdown of the plant. No unreviewed safety or environmental questions are involved.
1SC-1306	This change involves the structural work required to restrict unauthorized access through a roof opening by welding bars to the roof opening frame. The work area is the hallway between the Aux. Building and the A Building (122' elevation). This hallway is not in a safety related area of the plant. No unreviewed safety or environmental questions are involved.

*DCR - Design Change Request

PSE&G SALEM GENERATING STATION
SAFETY RELATED WORK ORDER LOG

SALEM UNIT 1

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION	
8503191008	SMD	1	13 FAN COIL UNIT	
			FAILURE DESCRIPTION:	SERVICE WATER LINE BROKEN OFF UPSTREAM OF 13SW243 PLEASE REPAIR
			CORRECTIVE ACTION:	WELDED NIPPLE PERFORMED AN ACCEPTABLE PT & IN SERV ICE LEAK EXAM
8503120950	SMD	1	11BF19	
			FAILURE DESCRIPTION:	REPAIR 11BF19 PACKING LEAK USING FURMANITE PROCESS.
			CORRECTIVE ACTION:	VALVE FURMANITED
8503121964	SIC	1	SOLENOID VALVE	
			FAILURE DESCRIPTION:	THE INLET VALVE TO THE RIGHT CHAMBER IS BLOWING AIR. POSSIBLE DIAPHRAGM LEAK. VALVE LOCATED UNDER CHAMBER.
			CORRECTIVE ACTION:	REPLACED VALVE AND MOISTURE INDICATOR.
8503070359	SIC	1	AFD MONITOR	
			FAILURE DESCRIPTION:	AFD RECORDER IS SPIKING LOW INTO ALARM. I&C DEPT PLEASE REPAIR.
			CORRECTIVE ACTION:	SWITCH #1 WAS REMOVED CONTACTS CLEANED AND RETURNED TO SERVICE. PROCEDURE 1PD-16.1.007 WAS USED TO REMOVE SWITCH FROM SERVICE TO WORK ON IT.

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION
8503070600	SIC	1	CONTROL FOR 11SW129
			FAILURE DESCRIPTION: SOLENOID VALVE IS CONTINUOUSLY VENTING
			CORRECTIVE ACTION: REBUILD SV-601 WITH ASCO SOLENOID REPAIR KIT.
0099160501	SMD	1	12SW24 12 STRNR. BK/WASH
			FAILURE DESCRIPTION: 12SW24 12SWP STRNR. BACKWASH VALVE, HAS BLOWN DIAPHRAGM
			CORRECTIVE ACTION: FOUND VALVE DIAPHRAGM BLOWN, AND INTERNALS DISTROYED. REPLACED WITH A NEW COMPLETE VALVE. NEW NUTS AND SUTDS. CLEANED AREA.
0099160676	SIC	1	VCT
			FAILURE DESCRIPTION: A SWAGELOCK FITTING ON LEVEL POT BOTTOM BELLOWS IS BLOWING CAUSING A RADIATION PROBLEM. PLEASE REPAIR
			CORRECTIVE ACTION: REPLACED FITTING SENSING LINE FOR LEVEL BELLOWS ON VCT.
0099160714	SMD	1	#13 CFCU MOTOR COOLER
			FAILURE DESCRIPTION: SERVICE WATER LEAK IN MOTOR COOLER, PLEASE REPAIR.
			CORRECTIVE ACTION: REPLACED MOTOR COOLER
0099160480	SMD	1	16 SW PMP STRNR.
			FAILURE DESCRIPTION: 16 SW PMP. STRNR. HAS A PACKING LEAK.
			CORRECTIVE ACTION: REPACKED STRAINER

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION
0099005255	SMD	1	12 BF19

FAILURE DESCRIPTION: PACKING GLAND LEAKING BADLY. PACKING HAS BEEN TIGHTENED AS MUCH AS POSSIBLE WORK ORDER TO COVER TEMP. REPAIRS, FURMANITE

CORRECTIVE ACTION: DRILLED TRAPPED AND INSERTED FURMANITE FITTINGS INJECTED FURNMANITE COMPOUND ASSURED VALVE STEM MOVEMENT NEW WO ISSUED FOR PERMANENT REPAIR IS 8502280201

SALEM GENERATING STATION
MONTHLY OPERATING SUMMARY - UNIT NO. 1
MARCH 1985

SALEM NO. 1

The Unit began the period operating at full power. On 3/07/85 at 0420 hours Unit power was reduced to 79% as a result of 12E Intercept valve failing to reopen after completion of a routine turbine valve surveillance test. The valve failed to reopen as a result of a cut seat on the hydraulic dump valve. At 1657 hours, with the completion of repairs, the Unit was returned to 100% power where it remained for the rest of the period. On March 31, 1985, No. 1 Unit's consecutive days in operation exceeded its previous record of 88 days.

REFUELING INFORMATION

DOCKET NO.: 50-272
COMPLETED BY: J. Ronafalvy
UNIT NAME: Salem 1
DATE: April 10, 1985
TELEPHONE: 609/935-6000
EXTENSION: 4455

Month March 1985

1. Refueling information has changed from last month:
   YES _____ NO X

2. Scheduled date for next refueling: February 22, 1986

3. Scheduled date for restart following refueling: May 4,1986

4. A) Will Technical Specification changes or other license amendments be required?
   YES _____ NO _____
   NOT DETERMINED TO DATE _____

   B) Has the reload fuel design been reviewed by the Station Operating Review Committee?
   YES _____ NO X
   If no, when is it scheduled? January 1986

5. Scheduled date(s) for submitting proposed licensing action:
   January 1986 if required

6. Important licensing considerations associated with refueling:
   NONE

7. Number of Fuel Assemblies:
   A) Incore 193
   B) In Spent Fuel Storage 296

8. Present licensed spent fuel storage capacity: 1170

   Future spent fuel storage capacity: 1170

9. Date of last refueling that can be discharged to spent fuel pool assuming the present licensed capacity: September 2001

8-1-7.R4

PSE&G

Public Service Electric and Gas Company P.O. Box E Hancocks Bridge, New Jersey 08038

Salem Generating Station

April 10, 1985

Director, Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Dear Sir:

MONTHLY OPERATING REPORT
SALEM NO. 1
DOCKET NO. 50-272

In Compliance with Section 6.9, Reporting Requirements for the Salem Technical Specifications, 10 copies of the following monthly operating reports for the month of March 1985 are being sent to you.

Average Daily Unit Power Level
Operating Data Report
Unit Shutdowns and Power Reductions
Major Plant Modification
Safety Related Work Orders
Operating Summary
Refueling Information

Sincerely yours,

J. M. Zupko, Jr.
General Manager - Salem Operations

JR:sbh
cc: Dr. Thomas E. Murley
Regional Administrator USNRC
Region I
631 Park Avenue
King of Prussia, PA 19406

Director, Office of Management
Information and Program Control
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Enclosures
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