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Gary J. Taylor Vice President **Nuclear Operations** 

July 10. 1996 RC-96-0150

Document Control Desk U.S. Nuclear Regulatory Commission Washington, DC 20555

Attention: Mr. A. R. Johnson

Gentlemen:

Subject:

VIRGIL C. SUMMER NUCLEAR STATION

DOCKET NO. 50/395

OPERATING LICENSE NO. NPF-12

SPECIAL REPORT (SPR 960004), REVISION 1

Reference: G. J. Taylor to Document Control Desk Letter (RC-96-0132), Dated

May 13, 1996

NOTE:

This letter supersedes the above referenced letter due to the inadvertent

omission of information regarding a tube imperfection in Steam Generator

"B".

This special report is being submitted pursuant to the requirements of the Virgil C. Summer Nuclear Station Technical Specifications, Section 4.4.5.5.a and Section 4.4.5.5.b.

Section 4.4.5.5.a requires that a Special Report be submitted, within 15 days following the completion of inservice inspection of Steam Generator tubes, indicating the number of tubes plugged or repaired in each generator. The inspection activity was completed on April 29, 1996. Summarized in the table below is the number of plugs installed in each generator as a result of the inspections performed during the ninth refueling outage.

Generator	A	В	C	Total
No. Plugged	0	0	0	0

Currently there are two plugs in Steam Generator "C" and one plug in Steam Generator "B" as a result of inspections performed prior to installing the replacement Steam Generators.

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Document Control Desk SPR 960004, Revision 1, RFL 300 RC-06-0150 Page 2 of 3

Section 4.4.5.5.b requires that within 12 months following the completion of the inservice inspection of the generators, the complete results be submitted to the Commission as a Special Report. This information is provided below:

1. The number and extent of the tubes inspected.

Steam Generator "A" - 1393 tubes (approximately 22%)

Steam Generator "B" - 1039 tubes (approximately 16%)

Together these comprise approximately 13% of the total tube population of the three Steam Generators. These tubes were inspected from tube end to tube end (full length) using the bobbin coil eddy current technique.

2. Location and percent of wall-thickness penetration for each indication of an imperfection.

A total of three tubes were identified with imperfections during the recent outage. All of these indications were present in the baseline examination performed in August 1994, prior to installation of the new generator and are less than 20% throughwall imperfections. These tubes are:

- Steam Generator "A" Row 113, Column 72, sixth support plate plus 10.09 inches on the outlet side. The imperfection depth was not quantifiable.
- Steam Generator "B" Row 52, Column 11, ninth support plate plus 1.09 inches on the outlet side. The imperfection depth is less than 20% of nominal wall thickness.
- Steam Generator "B" Row 65, Column 90, first support plate plus 29.86 inches on the outlet side. The imperfection depth is less than 20% of nominal wall thickness.
- 3. Identification of tubes plugged.

No tubes were plugged during this inspection interval.

VCSNS response to Generic Letter 95-03, dated February 1, 1996, stated that examination of 100 intersections will be done using 3-coil "plus point" MRPC techniques during the RF-9 inspection of our Steam Generators. The tubes selected for this additional inspection scope would be the ones exhibiting bobbin coil indications that may require further characterization. Since there were no new indications detected utilizing the bobbin coil technique, the scope was not expanded to include MRPC examinations.

Document Control Desk SPR 960004, Revision 1, RFL 300 RC-06-0150 Page 3 of 3

Should you have questions on the above, please call Mr. Philip Rose at (803) 345-4052.

Very truly yours,

Gary J

## PAR/GJT/dwr

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