

July 10, 1995

Mr. William R. McCollum
Site Vice President
Catawba Nuclear Station
Duke Power Company
4800 Concord Road
York, South Carolina 29745-9635

SUBJECT: CATAWBA NUCLEAR STATION, UNIT 2 - REQUEST FOR ADDITIONAL
INFORMATION - SECOND 10-YEAR INTERVAL INSERVICE INSPECTION PROGRAM
PLAN AND ASSOCIATED REQUESTS FOR RELIEF (TAC NO. M95255)

Dear Mr. McCollum:

The staff, with assistance from its contractor, Idaho National Engineering Laboratory, is reviewing and evaluating the second 10-year interval inservice inspection program plan and the associated requests for relief from the ASME B&PV Code, Section XI requirements for Catawba Nuclear Station, Unit 2. Additional information is needed from Duke Power Company in order for the staff to complete its review.

Please respond within 60 days to meet the staff's review schedule. In addition, to expedite the review process, please send a copy of your response to the staff's contractor, INEL, at the following address:

Michael T. Anderson
INEL Research Center
2151 North Boulevard
PO Box 1625
Idaho Falls, Idaho 83415-2209

Sincerely,

/s/

Peter S. Tam, Senior Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-414

Enclosure:
Request for additional information

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cc w/encl: See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

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Michael T. Anderson
INEL Research Center
2151 North Boulevard
PO Box 1625
Idaho Falls, Idaho 83415-2209

Sincerely,

A handwritten signature in dark ink, reading "Peter S. Tam". The signature is stylized with a large, sweeping "P" and "T".

Peter S. Tam, Senior Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-414

Enclosure:
Request for additional information

cc w/encl: See next page

Mr. W. R. McCollum
Duke Power Company

cc:

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Catawba Nuclear Station

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DUKE POWER COMPANY
CATAWBA NUCLEAR STATION, UNIT 2
DOCKET NUMBER 50-414

Request for Additional Information - Second 10-Year Interval Inservice Inspection (ISI) Program Plan

1. Scope/Status of Review

Throughout the service life of a water-cooled nuclear power facility, 10 CFR 50.55a(g)(4) requires that components (including supports) that are classified as American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code Class 1, Class 2, and Class 3 meet the requirements, except design and access provisions and preservice examination requirements, set forth in the ASME Code Section XI, "Rules for Inservice Inspection of Nuclear Power Plant Components," to the extent practical within the limitations of design, geometry, and materials of construction of the components. This section of the regulations also requires that inservice examinations of components and system pressure tests conducted during the successive 120-month inspection interval comply with the requirements in the latest edition and addenda of the Code incorporated by reference in 10 CFR 50.55a(b) on the date 12 months prior to the start of a successive 120-month interval, subject to the limitations and modifications listed therein. The components (including supports) may meet requirements set forth in subsequent editions and addenda of the Code that are incorporated by reference in 10 CFR 50.55a(b) subject to the limitations and modifications listed therein. The licensee, Duke Power Company, has prepared the *Catawba Nuclear Station, Unit 2, Second 10-Year Interval Inservice Inspection Program Plan* to meet the requirements of the 1989 Edition of Section XI of the ASME Code.

As required by 10 CFR 50.55a(g)(5), if the licensee determines that certain Code examination requirements are impractical and requests relief, the licensee shall submit information to the Nuclear Regulatory Commission (NRC) to support that determination. The staff has reviewed the available information in the *Catawba Nuclear Station, Unit 2, Second 10-Year Interval Inservice Inspection Program Plan*, Revision 0, submitted by letter dated February 14, 1996, and the requests for relief from the ASME Code Section XI requirements that the licensee has determined to be impractical.

2. Additional Information Required

Based on the above review, the staff has concluded that additional information and/or clarification is required to complete the review of the ISI Program Plan.

A. Address the degree of compliance and/or exceptions to augmented examinations that have been established by the NRC when added assurance of structural reliability is deemed necessary. Examples of documents that address augmented examinations are:

- (1) Branch Technical Position MEB 3-1, "High Energy Fluid Systems, Protection Against Postulated Piping Failures in Fluid Systems Outside Containment";
- (2) Regulatory Guide 1.150, *Ultrasonic Testing of Reactor Vessel Welds During Preservice and Inservice Examinations*;
- (3) Code of Federal Regulations, Part 10, 50.55a(g)(6)(ii)(A), which states that all licensees must augment their reactor vessel examinations by implementing once, during the inservice inspection interval in effect on September 8, 1992, the examination requirements for reactor vessel shell welds specified in Item B1.10 of Examination Category B-A of the 1989 Code. In addition, all previously granted relief for Item B1.10, Examination Category B-A, for the interval in effect on September 8, 1992, is revoked by the new regulation. For licensees with fewer than 40 months remaining in the interval on the effective date, deferral of the augmented examination is permissible with the conditions stated in the regulations.

Discuss these and any other augmented examinations that may have been incorporated in the *Catawba Nuclear Station, Unit 2, Second Ten-Year Inspection Interval Inservice Inspection Program*, Revision 0.

- B. Paragraph 10 CFR 50.55a(b)(2)(iv) requires that certain ASME Code Class 2 piping welds in the Residual Heat Removal (RHR), Emergency Core Cooling (ECC), and Containment Heat Removal (CHR) systems be examined. Portions of these systems are critical to the safe shutdown of the plant and should not be completely excluded from inservice volumetric examination based on piping wall thickness. In consideration of the safety significance of the subject systems, discuss any plans or schedules for the examination of a sample of these welds to assure the continued integrity of thin-wall piping. (A 7-1/2% sample is consistent with the extent of examination required for Class 2 piping.)
- C. The licensee has provided tables of the scheduled examinations for the second interval, by outage. However, in order to verify that the appropriate percentage of examination areas are being scheduled by period and for the interval, the total number of examination areas, summarized by Examination Category and the number of examination areas scheduled for each period and for the

interval is required. Provide a summary of the total number of examination areas, by Examination Category, and the number of examinations scheduled for each period and for the interval. This information will aid in the verification that the program schedule is in compliance with Code requirements.