AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-285
UNIT Fort Calhoun Station
DATE March 6, 1985
COMPLETED BY T. P. Matthews
TELEPHONE (402) 536-4733

MONTH February, 1985

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	486.5	17	486.9
2	486.5	18	486.6
3	486.0	19	487.1
4	486.6	20	487.1
5	487.4	21	487.7
6	486.8	22	487.8
7	487.0	23	488.1
8	486.7	24	487.6
9	486.5	25	487.4
10	486.0	26	487.0
11	486.6	27	486.7
12	487.0	28	486.7
13	486.8	29	
14	486.8	30	
15	487.1	31	
16	487.3		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

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OPERATING DATA REPORT

DOCKET NO. 50-285
DATE March 6, 1985
COMPLETED BY T. P. Matthews
TELEPHONE (402) 536-4733

OPERATING STATUS

1. Unit Name: Fort Calhoun Station
2. Reporting Period: February, 1985
3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 502
5. Design Electrical Rating (Net MWe): 478
6. Maximum Dependable Capacity (Gross MWe): 502
7. Maximum Dependable Capacity (Net MWe): 478
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
   Items 4 and 6 changed to be consistent with actual nameplate rating.

Notes

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: None

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	672.0	1,416.0	100,202.0
12. Number Of Hours Reactor Was Critical	672.0	1,416.0	76,696.2
13. Reactor Reserve Shutdown Hours	0.0	0.0	1,309.5
14. Hours Generator On-Line	672.0	1,416.0	76,083.4
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	1,001,136.2	2,109,431.1	96,296,198.1
17. Gross Electrical Energy Generated (MWH)	342,028.0	719,402.0	31,489,027.0
18. Net Electrical Energy Generated (MWH)	327,217.9	687,765.7	30,099,403.0
19. Unit Service Factor	100.0	100.0	75.9
20. Unit Availability Factor	100.0	100.0	75.9
21. Unit Capacity Factor (Using MDC Net)	101.9	101.6	65.4
22. Unit Capacity Factor (Using DER Net)	101.9	101.6	63.1
23. Unit Forced Outage Rate	0.0	0.0	3.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
    None

25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A
26. Units In Test Status (Prior to Commercial Operation): N/A

	Forecast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

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DOCKET NO. 50-285
UNIT NAME Fort Calhoun Station
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UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH February, 1985

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
									There were no unit shutdowns or power reductions during the month of February, 1985.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

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Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending February, 1985 .

1. Scheduled date for next refueling shutdown. October, 1985

2. Scheduled date for restart following refueling. December, 1985

3. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? Yes

   a. If answer is yes, what, in general, will these be?

      Technical Specification change to accommodate increased radial peaks due to further reduction in radial leakage.

   b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. _____

   c. If no such review has taken place, when is it scheduled? _____

4. Scheduled date(s) for submitting proposed licensing action and support information. September, 1985

5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

   Methodology Changes June, 1985

6. The number of fuel assemblies:
   a) in the core 133 assemblies
   b) in the spent fuel pool 305 "
   c) spent fuel pool storage capacity 729 "
   d) planned spent fuel pool storage capacity May be increased via fuel pin consolidation "

7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. 1996

Prepared by J K Gasper Date March 4, 1985

OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

February, 1985
Monthly Operations Report

I. OPERATIONS SUMMARY

Fort Calhoun Station operated at 100% throughout February, 1985.

During the month of February, three Auxiliary Operator-Nuclear personnel reported for work at Fort Calhoun Station.

The security force will be at full authorized strength in the first week of March.

The Chemistry/Radiation Protection group participated with Technical Services and Licensing in responding to questions by the NRC concerning steam generator secondary chemistry. The primary and secondary chemistry parameters were maintained within limits as to require no corrective actions.

The Maintenance and Technical groups continued support functions to maintain 100% power operation.

The Training Department continues to implement revised training programs. The initial course for new operators is scheduled to begin in mid-March.

No safety valve or PORV challenges or failures occurred.

A. PERFORMANCE CHARACTERISTICS

LER Number	Deficiency
84-023 R1	VIAS Actuations -- R1 corrected typos and clarified content of certain statements.

A. PERFORMANCE CHARACTERISTICS (continued)

LER Number	Deficiency
84-021	Low Boron Concentration in Safety Injection and Refueling Water Tank

B. CHANGES IN OPERATING METHODS

None

C. RESULTS OF SURVEILLANCE TESTS AND INSPECTIONS

None

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

Procedure	Description
SP-FAUD-1	Fuel Assembly Uplift Condition Detection. This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 since it only involved the evaluation of data from a surveillance test to verify that a fuel assembly uplift condition did not exist.

System Acceptance Committee Packages for February, 1985:

Package	Description/Analysis
EEAR FC-83-38	Installation of CRT Cables and Conduits. This modification provided for the installation of cable and conduit to selected locations for CRT hookup and has no adverse effect on the safety analysis.
EEAR FC-84-208	Condensate Sample Point Upstream of "A" Drain Cooler. This modification provided for the installation of a sample valve on the condensate system which already has an isolation valve. This modification has no adverse effect on the safety analysis.

D. CHANGES, TEST AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL (continued)

System Acceptance Committee Packages for February, 1985: (continued)

Package	Description/Analysis
EEAR FC-83-120	Gai-tronics for Office Area. This modification provided for the installation of two Gai-tronics stations in the office area and has no adverse effect on the safety analysis.
EEAR FC-83-24	Cutout Line Between RW-153 and RW-167. This modification provided for the removal of some unused piping and capping of the ends. This provided new drain connections, but did not change the raw water system performance or function. This modification has no adverse effect on the safety analysis.
DCR 76-66	Hazardous Material Storage Building. This modification provided for the construction and erection of a 20' x 30' preengineered metal building approximately 550' west and 150' north of the reactor containment centerline. This modification has no adverse effect on the safety analysis.
EEAR FC-84-85	Inlet Union for NG-123. This modification provided for the installation of a union on the inlet of NG-123 for ease of maintenance on the nitrogen gas system and has no adverse effect on the safety analysis.

E. RESULTS OF LEAK RATE TESTS

None

F. CHANGES IN PLANT OPERATING STAFF

During February, Messrs. Dennis Peters, Patrick Cronin and Joe Braun reported to the Fort Calhoun Station as Auxiliary Operators-Nuclear. Effective February 16, 1985, Mr. John J. Fluehr, III was appointed as Supervisor-Station Training.

G. TRAINING

Training in February, 1985, included operator requalification including fire brigade leader training for licensed operators, introduction to performance-based training and evaluation, and emergency procedures guideline review and development. Non-licensed operator training included classical physics, reactor theory and initial fire brigade training. Maintenance received systems training on the reactor coolant system and the demineralized water system.

H. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

None

II. MAINTENANCE (Significant Safety Related)

None

Alan W. Richard
for W. Gary Gates
Manager
Fort Calhoun Station

Omaha Public Power District
1623 Harney Omaha, Nebraska 68102
402/536-4000

March 14, 1985
LIC-85-103

Mr. James M. Taylor, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, DC 20555

Reference: Docket No. 50-285

Dear Mr. Taylor:

February Monthly Operating Report

Please find enclosed ten (10) copies of the February 1985 Monthly Operating Report for the Fort Calhoun Station Unit No. 1.

Please note the change to lines 4 and 6 of the Operating Data Report. The Nameplate Rating and Maximum Dependable Capacity figures were revised from 501 to 502 to reflect the correct values. This change does not affect the Monthly Operating Data that has been reported previously.

Sincerely,

R. L. Andrews
Division Manager
Nuclear Production

RLA/TPM/dao

Enclosures

cc: NRC Regional Office
Office of Management & Program Analysis (2)
Mr. R. R. Mills - Combustion Engineering
Mr. T. F. Polk - Westinghouse
Nuclear Safety Analysis Center
INPO Records Center
American Nuclear Insurers
NRC File

DESIGNATED ORIGINAL
Certified By 04/09/85

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