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the southern electric system

July 3, 1996

LCV-0835

Docket No. 50-424

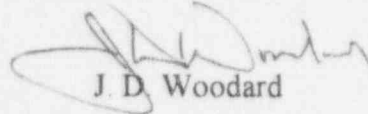
U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D. C. 20555

Ladies and Gentlemen:

VOGTLE ELECTRIC GENERATING PLANT
LICENSEE EVENT REPORT 1-96-7
MAIN STEAMLINE ISOLATED WHEN STEAM DUMP VALVE FAILED OPEN

In accordance with the requirements of 10 CFR 50.73, Georgia Power Company (GPC) hereby submits the enclosed report for Vogtle Electric Generating Plant associated with an event that occurred on June 12, 1996.

Sincerely,



J. D. Woodard

JDW/TEW/AFS

Enclosure: LER 1-96-7

cc: Georgia Power Company
Mr. J. B. Beasley, Jr.
Mr. M. Sheibani
NORMS

U. S. Nuclear Regulatory Commission
Mr. S. D. Ebnetter, Regional Administrator
Mr. L. L. Wheeler, Licensing Project Manager, NRR
Mr. C. R. Ogle, Senior Resident Inspector, Vogtle

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LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS MANDATORY INFORMATION COLLECTION REQUEST 50.0 HRS. REQUIRED LESSONS LEARNED ARE INCORPORATED INTO THE LICENSING PROCESS AND FED BACK TO INDUSTRY FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (T-6 F33), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001 AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503

FACILITY NAME (1)

Vogtle Electric Generating Plant - Unit 1

DOCKET NUMBER (2)

5 0 0 0 4 2 4 1 OF 3

PAGE (3)

TITLE (4)

MAIN STEAMLINES ISOLATED WHEN STEAM DUMP VALVE FAILED OPEN

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER(S)
0	6	1	2	9	6	9	6	0	0	7
0	6	1	2	9	6	9	6	0	0	7
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)							
3			20 2201(b) 20 2203(a)(2)(v) 50 73(a)(2)(i) 50 73(a)(2)(viii)							
POWER LEVEL (10)			20 2203(a)(1) 20 2203(a)(3)(i) 50 73(a)(2)(ii) 50 73(a)(2)(x)							
0			20 2203(a)(2)(i) 20 2033(a)(3)(ii) 50 73(a)(2)(iii) 73.71							
			20 2203(a)(2)(ii) 20 2033(c)(1) X 50 73(a)(2)(iv) OTHER							
			20 2203(a)(2)(iii) 50 36(c)(1) 50 73(a)(2)(v) Specify in Abstract below							
			20 2203(a)(2)(iv) 50 36(c)(2) 50 73(a)(2)(vii) or in NRC Form 366A							

NAME

Mehdi Sheibani, Nuclear Safety and Compliance

TELEPHONE NUMBER (include area code)

AREA CODE 7 0 6 8 2 6 - 3 2 0 9

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
B	S	B	P	C	V	B	0	4	5
				N					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
X					

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-space typewritten lines) (16)

On June 12, 1996, while in Mode 3 (hot standby), a manual isolation of the main steamlines was performed to limit a reactor coolant system cooldown initiated when a steam dump valve failed open. Reactor coolant system temperature dropped approximately 5 degrees F during this event. This event represents an unplanned ESF actuation.

The cause of the event was the failure of the steam dump valve to the full open position. Troubleshooting of the valve positioner found that a cam had loosened on its camshaft, eventually resulting in a rounding of the shaft, an inability of the shaft to turn the cam, and the cam's failure to close the valve on demand. The valve was repaired and returned to service.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

FACILITY NAME (1) Vogtle Electric Generating Plant - Unit 1	DOCKET NUMBER (2) 05000424	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		96	007	00	2	OF	3

TEXT (If more space is required, use additional copies of NRC Form 366A)(17)

A. REQUIREMENT FOR REPORT

This report is required per 10 CFR 50.73 (a)(2)(iv) because an unplanned engineered safety feature actuation occurred when the main steam lines were manually isolated.

B. UNIT STATUS AT TIME OF EVENT

At the time of this event, Unit 1 was operating in Mode 3 (hot standby) at 0 percent of rated thermal power and at normal operating temperature and pressure of 557 degrees F and 2235 psig. Other than that described herein, there was no inoperable equipment that contributed to the occurrence of this event.

C. DESCRIPTION OF EVENT

On June 12, 1996, shutdown rod banks had been withdrawn from the reactor and personnel were preparing to perform a reactor coolant system (RCS) leakrate surveillance test. The steam dump valves were in automatic mode, controlling RCS temperature. At 1435 EDT, control room personnel received a red light indication that steam dump valve 1PV-507B was open. When it failed to automatically reclose, the steam generator water level control operator (SGWLC) attempted to close the valve from the control room but was unable to close it. As RCS temperature began to drop, the unit shift supervisor directed personnel to isolate the main steamlines. The main and bypass isolation valves were closed at 1440 EDT, ending the transient. RCS temperature had dropped approximately 5 degrees F and the resultant decrease in pressurizer level caused an RCS letdown isolation, as expected.

D. CAUSE OF EVENT

The cause of this event was the failure of 1PV-507B to the full open position. Troubleshooting of the valve positioner found that a cam had loosened on its camshaft, eventually resulting in a rounding of the shaft, an inability of the shaft to turn the cam, and the cam's failure to close the valve on demand. The cause of the loosening of the cam could not be definitively determined.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Vogtle Electric Generating Plant - Unit 1	05000424	96	-007	-00	3	OF	3

TEXT (If more space is required, use additional copies of NRC Form 366A)(17)

E. ANALYSIS OF EVENT

When RCS temperature began to drop, control room personnel acted properly to isolate the main steamlines. The closing of the main steamline isolation valves prevented an uncontrolled cooldown of the RCS. There was no adverse affect on plant safety or on the health and safety of the public as a result of this event.

F. CORRECTIVE ACTION

- 1) Valve 1PV-507B was repaired and returned to service.
- 2) By July 19, 1996, all other steam dump valves on both Unit 1 and Unit 2 will be examined for potential similar failures.

G. ADDITIONAL INFORMATION

- 1) Failed Components:
Steam dump valve positioner manufactured by Bailey Controls
Type AP2
- 2) Previous Similar Events:
None
- 3) Energy Industry Identification System Code:
Main Steam System - SB
Reactor Coolant System - AB
Control Rod Drive System - AA