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John L. Skolds
Vice President
Nuclear Operations

October 6, 1992

Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, DC 20555

Attention: Mr. G. F. Wunder

Gentlemen:

Subject: VIRGIL C. SUMMER NUCLEAR STATION (VCSNS)
DOCKET NO. 50/395
OPERATING LICENSE NO. NPF-12
TECHNICAL SPECIFICATION CHANGE REQUEST - TSP 920002
INCREASE IN STEAM GENERATOR TUBE PLUGGING FROM 15% TO 18%

In accordance with 10CFR50.90, South Carolina Electric & Gas Company (SCE&G) is submitting an amendment request to License NPF-12 to amend the VCSNS Technical Specifications. The proposed change is a revision to Table 2.2-1, "Reactor Trip System Instrumentation Trip Setpoints," to allow an increase in the maximum permissible average level of Steam Generator Tube Plugging (SGTP) from 15% to 18%. An increase in SGTP reduces Reactor Coolant System Minimum Measured Flow (MMF) and, therefore, requires changes to a constant and a setpoint reduction penalty in the Overtemperature Delta T (OTΔT) setpoint equation, the OTΔT trip total allowance and Z, and the loop design flow listed in Table 2.2-1.

The change request is contained in the following attachments:

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|--------------|--|
| Attachment 1 | List of Affected Pages and Marked Up Technical Specifications |
| Attachment 2 | A Description of the Amendment Request and supporting Safety Evaluation |
| Attachment 3 | A Description of the Amendment Request and associated No Significant Hazards Determination |

As discussed with the NRC in the August 12, 1992, Steam Generator Replacement Project status meeting, SCE&G requires approval of this Technical Specification change by the start of the seventh refueling outage (scheduled for March 5, 1993). SCE&G requests a sixty day implementation period for this Technical Specification change to allow for necessary procedure revisions.

This proposed Technical Specification change has been reviewed and approved by the Plant Safety Review Committee and the Nuclear Safety Review Committee.

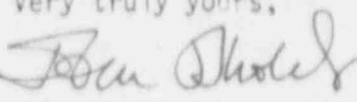
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I declare that the statements and matters set forth herein are true and correct to the best of my knowledge, information, and belief.

Should you have any questions concerning this issue, please call Ms. April R. Rice at (803) 345-4232 at your convenience.

Very truly yours,

John L. Skolds

ARR:lcd
Attachments

c: O. W. Dixon (w/o Attachments)
R. R. Mahan (w/o Attachments)
R. J. White
S. D. Ebnetter
G. F. Wunder
General Managers
NRC Resident Inspector
J. B. Knotts Jr.
H. G. Shealy
L. R. Cartin
RTS (TSP 920002)
File (813.20)

Attachments to Document Control Desk Letter

Technical Specification Change Request - TSP 920002

Increase in Steam Generator Tube Plugging
from 15% to 18%

ABBREVIATIONS

ASME	American Society of Mechanical Engineers
BIT	Boron Injection Tank
COLR	Core Operating Limits Report
CRDM	Control Rod Drive Mechanism
DNB	Departure from Nucleate Boiling
DNBR	Departure from Nucleate Boiling Ratio
ECCS	Emergency Core Cooling System
EOP	Emergency Operating Procedure
ECT	Eddy Current Testing
F Δ H	Hot Channel Enthalpy Rise Factor
F _Q	Total Peaking Factor
FSAR	Final Safety Analysis Report
GPM	Gallons per Minute
LBB	Leak Before Break
LBLOCA	Large Break Loss of Coolant Accident
LOCA	Loss of Coolant Accident
LOL/TT	Loss of Load/Turbine Trip
M/E	Mass and Energy
MMF	Minimum Measured Flow
MSSV	Main Steam Safety Valve
NRC	Nuclear Regulatory Commission
NSSS	Nuclear Steam Supply System
OP Δ T	Overpower Delta T
OT Δ T	Overtemperature Delta T
PCT	Peak Clad Temperature
PLOF	Partial Loss of Flow
RC	Reactor Coolant
RCCA	Rod Cluster Control Assembly
RCP	Reactor Coolant Pump
RCS	Reactor Coolant System
RHR	Residual Heat Removal
RSR	Relative Stability Ratio
RTP	Rated Thermal Power
RWST	Refueling Water Storage Tank
SBLOCA	Small Break Loss of Coolant Accident
SCE&G	South Carolina Electric & Gas Company
SIS	Safety Injection System
S/G	Steam Generator
SGTP	Steam Generator Tube Plugging
SGTR	Steam Generator Tube Rupture
TA	Total Allowance
TAVG	RCS Average Temperature
THOT	Vessel Outlet Temperature
TCOLD	Vessel Inlet Temperature
TDF	Thermal Design Flow
VCSNS	Virgil C. Summer Nuclear Station