



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

May 30, 1979

Docket No. 50-219

Mr. I. R. Finfrock, Jr.
Vice President - Generation
Jersey Central Power & Light Company
Madison Avenue at Punch Bowl Road
Morristown, New Jersey 07960

Dear Mr. Finfrock:

We have completed our review of the May 2, 1979 transient at the Oyster Creek Nuclear Generating Station that resulted in the reactor water level decreasing below the low-low-low level, which was set at 4 feet 8 inches above the top of the active fuel. Enclosed is our Safety Evaluation Report that presents our review and conclusions regarding (1) the event, (2) the condition of the core, and (3) the corrective action taken to prevent recurrence.

During your review and formulation of corrective action to prevent recurrence you proposed two changes to the Technical Specifications. These changes are to modify Section 2.1.D to clearly define the low-low-low water level as a safety limit for all modes of operation and to add Section 2.1.F containing a second safety limit which requires at least two recirculation loops to have the pump suction and discharge valves in the full open position at all times except when the reactor head is removed. Implementation of these two technical specification changes is a prerequisite to resuming operation. This license amendment with its related Safety Evaluation Report will be transmitted to you as a separate action.

As a result of your analysis "Bounding Loss of Coolant Inventory Transient for the Oyster Creek Plant," dated May 19, 1979, you agreed to propose another Technical Specification change. This change would add a limiting safety system setting to Section 2.3 of the Specifications to require automatic initiation of the isolation condenser on a low-low reactor vessel water level signal. This requirement would provide automatic corrective action to prevent the safety limits on reactor vessel level from being exceeded. This license amendment must be submitted by June 1, 1979. We will evaluate and act on that request as a separate issue as soon as it is received.

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Mr. I. R. Finfrock

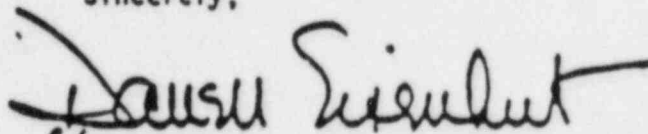
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We suggest that you consider other means of supplying high pressure water to the primary system in event of abnormal conditions. The need for additional cooling capabilities and the review of any modification you propose will be evaluated in the context of the Systematic Evaluation Program.

Based on our review of the May 2, 1979 event, the information on the condition of the core, and the corrective action taken (including the changes to the Technical Specifications) to prevent recurrence, we have concluded that there is reasonable assurance that there was no damage to the core and there is reasonable assurance that the health and safety of the public will not be endangered by resumption of operation. Therefore, pursuant to 10 CFR 50.36 (c)(1) the Oyster Creek Nuclear Generating Station is authorized to resume operation.

Sincerely,



Victor Stello, Jr., Director
Division of Operating Reactors

Enclosure:
Safety Evaluation
Report

cc w/enclosures:
See next page

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Mr. I. R. Finrock, Jr.

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