

LICENSEE EVENT REPORT (LER)

| | | |
|---|--|----------------------|
| FACILITY NAME (1) Browns Ferry - Unit 1 | DOCKET NUMBER (2) 0 5 0 0 0 2 5 9 1 | PAGE (3) 1 OF 0 2 |
|---|--|----------------------|

TITLE (4)
Reactor Core Isolation Cooling Controller Inoperable

| EVENT DATE (5) | | | LER NUMBER (6) | | | REPORT DATE (7) | | | OTHER FACILITIES INVOLVED (8) | | | | | | | | | | | | | | | |
|----------------|-----|------|----------------|-------------------|-----------------|-----------------|-----|------|-------------------------------|---|------------------|---|---|---|---|---|---|---|---|---|---|---|---|---|
| MONTH | DAY | YEAR | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | MONTH | DAY | YEAR | FACILITY NAMES | | DOCKET NUMBER(S) | | | | | | | | | | | | | |
| 1 | 2 | 1 | 9 | 8 | 4 | 8 | 4 | 0 | 0 | 3 | 8 | 0 | 0 | 0 | 1 | 1 | 8 | 8 | 5 | 0 | 5 | 0 | 0 | 0 |

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|--------------------------------|--|--|------------------|-----------------|--|-----------------|--|--|--|--|--|--|
| OPERATING MODE (9) N | THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11) | | | | | | | | | | | |
| | 20.402(b) | | | 20.406(c) | | | 50.73(a)(2)(iv) | | | 73.71(b) | | |
| | 20.406(a)(1)(i) | | | 50.38(c)(1) | | | <input checked="" type="checkbox"/> 50.73(a)(2)(v) | | | 73.71(c) | | |
| | 20.406(a)(1)(ii) | | | 50.38(c)(2) | | | 50.73(a)(2)(vii) | | | OTHER (Specify in Abstract below and in Text, NRC Form 366A) | | |
| | 20.406(a)(1)(iii) | | | 50.73(a)(2)(i) | | | 50.73(a)(2)(viii)(A) | | | | | |
| | 20.406(a)(1)(iv) | | | 50.73(a)(2)(ii) | | | 50.73(a)(2)(viii)(B) | | | | | |
| 20.406(a)(1)(v) | | | 50.73(a)(2)(iii) | | | 50.73(a)(2)(ix) | | | | | | |
| POWER LEVEL (10) 1 0 0 | | | | | | | | | | | | |

LICENSEE CONTACT FOR THIS LER (12)

| | |
|-------------------------------|---|
| NAME David L. Smith | TELEPHONE NUMBER AREA CODE 2 0 5 7 2 9 - 3 8 6 5 |
|-------------------------------|---|

| COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) | | | | | | | | | | |
|--|--------|-----------|--------------|---------------------|-------|--------|-----------|--------------|---------------------|---|
| CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NPROS | CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NPROS | |
| X | B | N | C | N | V | W | 2 | 9 | 0 | Y |

| | | | | | | |
|---|--|--|-------------------------------|-------|-----|------|
| SUPPLEMENTAL REPORT EXPECTED (14) | | | EXPECTED SUBMISSION DATE (15) | MONTH | DAY | YEAR |
| <input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE) <input checked="" type="checkbox"/> NO | | | | | | |

ABSTRACT (Limit to 1400 spaces, i.e. approximately fifteen single space typewritten lines) (16)

During performance of Surveillance Instructions 4.5.F.1.b, the Reactor Core Isolation Cooling (RCIC) turbine failed to respond above 2100 revolutions per minute. Therefore, it was declared inoperable. Technical Specification requirements were met. The RCIC ramp generator and signal converter was replaced, RCIC was successfully retested and declared operable. This failure was random with no further corrective action required.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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|--|--|----------------|-------------------|-----------------|----------|----|-----|
| FACILITY NAME (1) Browns Ferry - Unit 1 | DOCKET NUMBER (2) 0 5 0 0 0 2 5 9 8 4 | LER NUMBER (6) | | | PAGE (3) | | |
| | | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | | | |
| | | - 0 3 8 | - 0 0 | | 0 2 | OF | 0 2 |

TEXT (If more space is required, use additional NRC Form 366A's) (17)

During normal operation, unit 1 was operating at 100 percent power, unit 3 was operating at 44 percent, and unit 2 was in a refueling outage.

As surveillance test 4.5.F.1.b was being run on the Reactor Core Isolation Cooling System (BN), manual control signals failed to cause proper turbine (TRB) responses. Specifically, the turbine revolutions per minute would not respond above 2100. The Reactor Core Isolation Cooling System was declared inoperable. Technical Specification requirements were immediately initiated.

Within 24 hours, the Reactor Core Isolation Cooling System was repaired and successfully retested. The ramp generator and signal converter (CNV) was replaced. Its output signals were a constant voltage rather than a ramp function. This was the only problem found. Since this was a random failure, no further corrective action is necessary.

With the Reactor Core Isolation Cooling System inoperable, adequate emergency core cooling was maintained (per the requirements of the Final Safety Analysis Report) by the High Pressure Coolant Injection System (BJ).

Responsible Plant Section - N/A

Previous Similar Events - BFRO 50-259/84036

TENNESSEE VALLEY AUTHORITY

Browns Ferry Nuclear Plant

P. O. Box 2000

Decatur, Alabama 35602

January 18, 1985

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

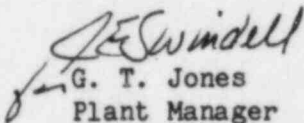
Dear Sir:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT (BFN) UNIT 1 -
DOCKET NO. 50-259 - FACILITY OPERATING LICENSE DPR-33 - REPORTABLE
OCCURRENCE REPORT BFRO-50-259/84038

The enclosed report provides details concerning reactor core isolation
cooling controller inoperability. This report is submitted in
accordance with 10 CFR 50.73 (a)(2)(v).

Very truly yours,

TENNESSEE VALLEY AUTHORITY



G. T. Jones
Plant Manager
Browns Ferry Nuclear Plant

Enclosures

cc (Enclosures):

Regional Administrator
U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region II
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INPO Records Center
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NRC Resident Inspector, BFN