Commonwealth Edison
Quad Cities Nuclear Power Station
22710 206 Avenue North
Cordova, Illinois 61242
Telephone 309/654-2241

GCT-92-021

May 1, 1992

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

SUBJECT: Quad Cities Nuclear Station Units 1 and 2
Monthly Performance Report
NRC Docket Nos. 50-254 and 50-265

Enclosed for your information is the Monthly Performance Report covering the operation of Quad-Cities Nuclear Power Station, Units One and Two, during the month of April 1992.

Respectfully,

COMMONWEALTH EDISON COMPANY
QUAD-CITIES NUCLEAR POWER STATION

R Waldorf

Gerald C. Tietz
Technical Superintendent

GCT/MB/dak

Enclosure

cc: A. B. Davis, Regional Administrator
T. Taylor, Senior Resident Inspector

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QUAD-CITIES NUCLEAR POWER STATION

UNITS 1 AND 2

MONTHLY PERFORMANCE REPORT

APRIL 1992

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS & ELECTRIC COMPANY

NRC DOCKET NOS. 50-254 AND 50-265

LICENSE NOS. DPR-29 AND DPR-30

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I. INTRODUCTION

Quad-Cities Nuclear Power Station is composed of two Boiling Water Reactor , each with a Maximum Dependable Capacity of 769 MWe Net, located in Cordova, Illinois. The Station is jointly owned by Commonwealth Edison Company and Iowa-Illinois Gas & Electric Company. The Nuclear Steam Supply Systems are General Electric Company Boiling Water Reactors. The Architect/Engineer was Sargent & Lundy, Incorporated, and the primary construction contractor was United Engineers & Constructors. The Mississippi River is the condenser cooling water source. The plant is subject to license numbers DPR-29 and DPR-30, issued October 1, 1971, and March 21, 1972, respectively; pursuant to Docket Numbers 50-254 and 50-265. The date of initial Reactor criticalities for Units One and Two, respectively were October 18, 1971, and April 26, 1972. Commercial generation of power began on February 18, 1973 for Unit One and March 10, 1973 for unit Two.

This report was compiled by Matt Benson and Debra Kelley, telephone number 309-654-2241 extensions 2995 and 2240.

II. SUMMARY OF OPERATING EXPERIENCE

A. Unit One

On April 5, Unit One dropped to 203 MWe for 1B Drywell Equipment Drain Sump Pump replacement. Load reductions occurred on April 7, to 320 MWe and April 10, to 278 MWe. Both of these drops were caused by the 1A recirculation pump tripping due to the turbine building 125 VDC 1A main bus trip. Another load reduction to 285 MWE was performed on April 19, for turbine testing. All other load reductions were requested by Chicago Load Dispatch.

B. Unit Two

Unit Two continued scheduled refuel outage.

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY RELATED MAINTENANCE

A. Amendments to Facility License or Technical Specifications

There were no Amendments to the Facility License or Technical Specifications for the reporting period.

B. Facility or Procedure Changes Requiring NRC Approval

There were no Facility or Procedure changes requiring NRC approval for the reporting period.

C. Tests and Experiments Requiring NRC Approval

There were no Tests or Experiments requiring NRC approval for the reporting period.

D. Corrective Maintenance of Safety Related Equipment

The following represents a tabular summary of the major safety related maintenance performed on Units One and Two during the reporting period. This summary includes the following: Work Request Numbers, Licensee Event Report Numbers, Components, Cause of Malfunctions, Results and Effects on Safe Operation, and Action Taken to Prevent Repetition.

UNIT 1 MAINTENANCE SUMMARY

WORK REQUEST	SYSTEM	EID DESCRIPTION	WORK PERFORMED
Q7398	2400	Repair bad cam amplifier card. Removed from 2-2406A, WRQ73834	Return to vendor for repair.
Q99447	5200	Repair leak in 1 inch flexible fuel oil supply line at south end of Diesel Generator.	As Found: Found hose with temporary repair. As Left: Replace hose and tightened connections.
Q99363	756	Investigate APRM #3. Has hi alarm and will not reset.	As Left: Installed spare flow control trip ref module 238X45862 and calibrated. Left APRM in bypass.
Q99753	1001	Investigate and repair minimum flow valve (transmitter diff press 1A RHR) which failed to open when A division pumps were discharging through B loop.	As Left: Install new aux contacts and applying grease, voltage was applied to both contactors. Readings were the same.
Q99482	1462	Repair stripped/leaking tubing fitting under CS pump disch press SW on pressure switch 1B core spray pump discharge.	Ferrel set fell apart when nut removed. Replaced with new length of tubing/fittings. Performed functional test to verify operability. No leaks seen.
Q99470	5206	Repair leak in diesel generator fuel oil piping.	As Found: Nipple loose. As Left: Replaced fuel pump discharge pipe and applied sealant.

UNIT 2 MAINTENANCE SUMMARY

WORK REQUEST	SYSTEM	EID DESCRIPTION	WORK PERFORMED
Q97165	20	Repair 2B RHR submarine door.	As Found: Hand would not latch and throw lever was bent. One lug bolt was not touching then hinge. The upper hinge pin torsional spring assembly was sticking. As Left: Adjusted the door latch handle.
Q99642	220	Repair solenoid reactor head seal leak off.	Tightened, adjusted and tested limits.
Q99724	902	Tighten or replace loose wire found on the field side of TBC-F4.	Removed wires from terminal points F3 and F4 and installed ring tounges then installed back to terminal points.
Q89476	2301	Repair valve drain pot outlet to an acceptable range for valve to close.	Installed new valve trim, stem and plug. Installed new diaphragm in valve operator.
Q99797	2540	Check operability of and/or repair and calibrate ACAD flow control and/or loop C of ACAD dilution flow controller.	Calibrated square root and flow indicator.
Q83162	756	Investigate and fix output voltage from high voltage LRPM 24-25 DET D power supply.	Fuse F10 blown. Replaced fuse F10.
Q85484	756	Investigate and repair LRPM 24-33C.	Bock tested under pot. Broke down at 80 VDC. Bad detector. Replaced detector and tested under WRQ93650.
Q88392	1360	Replace fittings, caps on RCIC high steam flow switch.	As Found: Test fittings stripped out on both the hi and low side of the ilstion manifold. Replaced fittings.
Q95830	5745	Repair U-2 2C RHRSW vault room cooler cooling coil crack.	As Found: Cap screws were broken on cover. 2 sheet metal screws were missing.

UNIT 2 MAINTENANCE SUMMARY

WORK REQUEST	SYSTEM	EID DESCRIPTION	WORK PERFORMED
Q97506	7800	Replace cracked insulation on drywell cooling blower 2A BKR transformer wires H-2, H-3.	Replaced transformer and performed EQ splices and taped and reinstalled breaker.
Q99278	1071	Rewire U2 RHR 1001-81A pressure transmitter.	As Found: White signal wire had approximately 1/8" of bare wire between insulation and end of terminal lug. As Left: Replaced terminal.
Q99287	6700	Replace lug on black connector of cable 28574 in cubicle 10 of bus 23 at terminal.	As Found: Wire was loose in lug. As Left: Replaced lug.
Q99436	1002	Repair cooling coil leak at upper motor bearing cooling line on motor 2C RHR pump.	As Found: 1 loose connection As Left: Retightened threaded connections.
Q99491	5746	Repair 2A RHR room cooler fan.	As Found: Thermostat set at 100, switch in auto, fan off. As Left: Turned the thermostat down to 78 and the fan came on. Repeated two more times. Set thermostat back to 100 degrees F.
Q99634	1601	2-1601-57 operates sporadically.	Greased moving parts of open contactor and a test switch to activate the open and closed contactors.
Q97507	7800	Replace 2D drywell cooling breaker transformer.	Replaced transformer.
Q98456	8000	Repair RPS MG breaker.	Obtained new breaker and undervoltage relay and tested it on BTS 500 breaker test rig.
Q98579	2300	Repair U2 HPCI emerg brg oil pump htr bkr.	Heater works fine. Reinstalled heater.
Q98715	260	Test flow converter APRM 4-6.	Installed new power supply and calibrated it along with existing flow converter as per QIP 756-5.
Q99030	2540	Repair SQ Rooter extractor on controller flow B ACAD vent line to SBGT.	Square rooter did not have any output. As Left: Removed square root extractor. Component and extractor worked properly.

UNIT 2 MAINTENANCE SUMMARY

WORK REQUEST	SYSTEM	EID DESCRIPTION	WORK PERFORMED
Q99030	2540	Repair SQ Rooter B ACAD vent flow.	Replaced with like component and extractor worked properly.
Q99030	2540	Repair SQ Rooter B ACAD vent flow indicator.	Replaced with like component and extract· .
Q99030	2540	Repair Transmitter flow ATM containment.	Removed Square rooter extractor. Replaced with like component and extractor.
Q99030	2540	Repair flow element ACAD B vent flow.	Removed Square root extractor. Replaced with like component and extractor.
Q99030	2540	Repair E/P ACAD vent to SBGT.	Removed Square root extractor. Replaced with like component and extractor.
Q99636	1002	Repair cooling piping for 2C RHR pump.	Repaired motor end of suction line flex hose had a crack all the way across flare at flexible swivel.
Q99302	500	Repair U2 Aux electric room EPA 2AB-2.	Found breaker operational. Removed deficiency tag.
Q99169	311	Repair leak in CRD withdrawal riser vent valve.	As Found: Bushing stripped out. As Left: Installed new tube adapter and quick disconnect.
Q9909171	311	Replace quick disconnect on CRD withdrawal riser vent valve.	As Found: Quick disconnect leaked. As Left: Replaced quick disconnect fitting.
Q99172	311	Replace quick disconnect on CRD insert riser vent valve.	As Found: Quick disconnect leaking. As Left: Replaced quick disconnect fitting.
Q99174	311	Install quick disconnect CRD insert riser vent valve.	As Found: Quick disconnect fitting missing. As Left: Installed quick disconnect fitting.

UNIT 2 MAINTENANCE SUMMARY

WORK REQUEST	SYSTEM	EID DESCRIPTION	WORK PERFORMED
Q87288	756	Investigate LPRM 40-09 detector and connector.	As Found: LPRM bypassed. As Left: Returned LPRM to operation.
Q89018	2303	Inspect/Test switch level U-2 HPCI gland seal hotwell pump.	Repaired alarm level switch and replaced gasket.
Q96874	2341	Repair wire on HPCI turbine thermocouple cables.	As Found: Splice exposed. As Left: Removed old tape. Rewrapped splices.
Q96618	1600	Repair dent in inner sealing rim of drywell hatch X-1.	Used a file and emery cloth to smooth the area.

IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for Quad-Cities Units One and Two occurring during the reporting period, pursuant to the reportable occurrence reporting requirements as set forth in sections 6.6.B.1 and 6.6.B.2 of the Technical Specifications.

UNIT 1

Licensee Event Report Number	Date	Title of Occurrence
92-009	04-01-92	Missed Tech Spec Functional Test requirements.
92-010	04-07-92	Loss of Unit 1 125 VDC Bus TB 1A.
92-012	04-22-92	Main Chimney Hi Range Noble Gas Monitor Inop 7 days.
92-013	04-23-92	Second level undervoltage relays setpoint nonconservative.
92-014	04-24-92	LPCI mode of RHR administratively Inop.
92-015	04-29-92	LPCI & C.S. valve yokes outside design basis.

UNIT 2

92-011	04-02-92	Loss of transformer 22
92-012	04-09-92	U2 RHR Support outside FSAR limits.
92-013	04-07-92	Second Level UV setpoint non conservative.
92-014	04-16-92	1/2 Diesel Generator Inop to Unit (outside design basis).

V. DATA TABULATIONS

The following data tabulations are presented in this report:

A. Operating Data Report

B. Average Daily Unit Power Level

C. Unit Shutdowns and Power Reductions

APPENDIX C
OPERATING DATA REPORT

DOCKET NO 50-254
UNIT One
DATE May 6, 1992
COMPLETED BY Matt Benson
TELEPHONE (309) 654-2241

OPERATING STATUS

1. REPORTING PERIOD: 0000 040192 2400 043092 GROSS HOURS IN REPORTING PERIOD: 720

2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 2511 MAX. DEPEND. CAPACITY: 769
DESIGN ELECTRICAL RATING (MWe-Net): 789

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A

4. REASONS FOR RESTRICTION (IF ANY):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	720.0	2631.30	139142.40
6. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	3421.9
7. HOURS GENERATOR ON LINE	720.0	2612.00	134843.10
8. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	909.2
9. GROSS THERMAL ENERGY GENERATED (MWH)	1738900.8	6337195.2	290387225.2
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)	567008.0	2072076.0	94146412.0
11. NET ELECTRICAL ENERGY GENERATED (MWH)	543120.0	2010150.0	88719518.0
12. REACTOR SERVICE FACTOR	100.00	90.61	79.16
13. REACTOR AVAILABILITY FACTOR	100.00	90.61	81.11
14. UNIT SERVICE FACTOR	100.00	89.94	76.71
15. UNIT AVAILABILITY FACTOR	100.00	89.94	77.23
16. UNIT CAPACITY FACTOR (Using MDC)	98.09	90.01	65.64
17. UNIT CAPACITY FACTOR (Using Design MWe)	95.61	87.73	63.97
18. UNIT FORCED OUTAGE RATE	0.0	13.06	5.81

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:
21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):

	FORECAST	ACHIEVED
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

APPENDIX C
OPERATING DATA REPORT

DOCKET NO 50-265
UNIT Two
DATE May 6, 1992
COMPLETED BY Kelt Benson
TELEPHONE (309) 654-2241

OPERATING STATUS

1. REPORTING PERIOD: 0000 040192 2400 043092 GROSS HOURS IN REPORTING PERIOD: 720

2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 2511 MAX. DEPEND. CAPACITY: 769
DESIGN ELECTRICAL RATING (MWe-Net): 789

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A

4. REASONS FOR RESTRICTION (IF ANY):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	0.0	0.25	133482.95
6. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	2985.8
7. HOURS GENERATOR ON LINE	0.0	0.25	130020.15
8. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	702.9
9. GROSS THERMAL ENERGY GENERATED (MWH)	0.0	175.2	280084062.20
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)	0.0	50.0	89930240.0
11. NET ELECTRICAL ENERGY GENERATED (MWH)	-537303.0	-553112.0	84581063.0
12. REACTOR SERVICE FACTOR	0.0	0.01	76.62
13. REACTOR AVAILABILITY FACTOR	0.0	0.01	78.33
14. UNIT SERVICE FACTOR	0.0	0.01	74.63
15. UNIT AVAILABILITY FACTOR	0.0	0.01	75.04
16. UNIT CAPACITY FACTOR (Using MDC)	-97.04	-24.77	63.13
17. UNIT CAPACITY FACTOR (Using Design MWe)	-94.58	-24.14	61.53
18. UNIT FORCED OUTAGE RATE	0.0	0.0	8.09

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:
21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):

	FORECAST	ACHIEVED
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

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APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO 50-254
UNIT One
DATE May 6, 1992
COMPLETED BY Matt Benson
TELEPHONE (309) 654-2241

MONTH April 1992

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	787	17.	784
2.	789	18.	785
3.	792	19.	652
4.	782	20.	780
5.	46	21.	782
6.	782	22.	768
7.	689	23.	753
8.	784	24.	783
9.	766	25.	783
10.	614	26.	751
11.	784	27.	783
12.	757	28.	783
13.	784	29.	761
14.	783	30.	780
15.	784	31.	
16.	783		

INSTRUCTIONS
On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.
These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO 50-265
UNIT Two
DATE May 6, 1992
COMPLETED BY Matt Benson
TELEPHONE (309) 654-2241

MONTH April 1992

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	-8	17.	-8
2.	-8	18.	-8
3.	-8	19.	-8
4.	-8	20.	-8
5.	-8	21.	-8
6.	-8	22.	-8
7.	-8	23.	-8
8.	-8	24.	-8
9.	-8	25.	-8
10.	-8	26.	-8
11.	-8	27.	-8
12.	-8	28.	-8
13.	-8	29.	-8
14.	-8	30	-8
15.	-8	31.	
16.	-8		

INSTRUCTIONS
On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.
These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-265

UNIT NAME Unit Two

COMPLETED BY Matthew Benson

DATE May 6, 1992

REPORT MONTH April 1992

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
92-01	4-1-92	S	720	C	4	- - - -	- -	- - - -	Continuation of Unit 2 Scheduled Refuel Outage.

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-254

UNIT NAME Unit 1

COMPLETED BY Matthew Benson

DATE May 6, 1992

REPORT MONTH April 1992

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
92-04	4-05-92	S	17.3	B	5				1B Drywell Equipment Drain Sump Pump Replacement
92-05	4-07-92	F	0.6	A	5	92-10			1A Recirc Pump Trip from 125 VDC 1A Main Bus Trip
92-06	4-10-92	F	6.5	A	5	92-10			1A Recirc Pump Trip from 125 VDC 1A Main Bus Trip
92-07	4-12-92	S	3.7	F	5				Reduced Load per Chicago L.D.
92-08	4-19-92	S	7.6	B	5				Turbine Testing
92-09	4-22-92	S	4.6	F	5				Reduced Load per Chicago L.D.
92-10	4-23-92	S	5.6	F	5				Reduced Load per Chicago L.D.
92-11	4-26-92	S	3.9	F	5				Reduced Load per Chicago L.D.
92-12	4-29-92	S	3.3	F	5				Reduced Load per Chicago L.D

VI. UNIQUE REPORTING REQUIREMENTS

The following items are included in this report based on prior commitments to the commission:

A. Main Steam Relief Valve Operations

There were no Main Steam Relief Valve Operations for the reporting period.

B. Control Rod Drive Scram Timing Data for Units One and Two

There was no Control Rod Drive scram timing data for Units One and Two for the reporting period.

VII. REFUELING INFORMATION

The following information about future reloads at Quad-Cities Station was requested in a January 26, 1978, licensing memorandum (78-24) from D. E. O'Brien to C. Reed, et al., titled "Dresden, Quad-Cities and Zion Station--NRC Request for Refueling Information", dated January 18, 1978.

QTP 300-S32
Revision 2
October 1989

QUAD CITIES REFUELING INFORMATION REQUEST

1. Unit: Q1 Reload: 11 Cycle: 12

2. Scheduled date for next refueling shutdown: 9-5-92

3. Scheduled date for restart following refueling: 12-5-92

4. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment:

   NOT AS YET DETERMINED.

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

   NOT AS YET DETERMINED.

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

   NONE AT PRESENT TIME.

7. The number of fuel assemblies.

   a. Number of assemblies in core: 724

   b. Number of assemblies in spent fuel pool: 1405

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:

   a. Licensed storage capacity for spent fuel: 3657

   b. Planned increase in licensed storage: 0

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2009

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OCT 30 1989
Q.C.O.S.R.

QTP 300-S32
Revision 2
October 1989

QUAD CITIES REFUELING INFORMATION REQUEST

1. Unit: Q2 Reload: 10 Cycle: 11

2. Scheduled date for next refueling shutdown: 01/01/92

3. Scheduled date for restart following refueling: 05/07/92

4. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment: Yes, as listed below:
   1. Remove Table 3.7-2
   2. Modification to turbine control valve fast acting solenoid valve.
   3. Modification to HPCI turbine exhaust steam line.
   4. HPCI/RCIC 24-hour shutdown action provision.

5. Scheduled date(s) for submitting proposed licensing action and supporting information:
   1. 01/15/92
   2. 04/18/91
   3. 06/28/91
   4. 12/31/91

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

   NONE AT PRESENT TIME.

7. The number of fuel assemblies.

   a. Number of assemblies in core: 724

   b. Number of assemblies in spent fuel pool: 2439

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:

   a. Licensed storage capacity for spent fuel: 3897

   b. Planned increase in licensed storage: 0

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2009

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OCT 30 1989
Q.C.O.S.R.

VIII. GLOSSARY

The following abbreviations which may have been used in the Monthly Report, are defined below:

ACAD/CAM - Atmospheric Containment Atmospheric Dilution/Containment Atmospheric Monitoring
ANSI - American National Standards Institute
APRM - Average Power Range Monitor
ATWS - Anticipated Transient Without Scram
BWR - Boiling Water Reactor
CRD - Control Rod Drive
EHC - Electro-Hydraulic Control System
EOF - Emergency Operations Facility
GSEP - Generating Stations Emergency Plan
HEPA - High-Efficiency Particulate Filter
HPCI - High Pressure Coolant Injection System
HRSS - High Radiation Sampling System
IPCLRT - Integrated Primary Containment Leak Rate Test
IRM - Intermediate Range Monitor
ISI - Inservice Inspection
LER - Licensee Event Report
LLRT - Local Leak Rate Test
LPCI - Low Pressure Coolant Injection Mode of RHRs
LPRM - Local Power Range Monitor
MAPLHGR - Maximum Average Planar Linear Heat Generation Rate
MCPR - Minimum Critical Power Ratio
MFLCPR - Maximum Fraction Limiting Critical Power Ratio
MPC - Maximum Permissible Concentration
MSIV - Main Steam Isolation Valve
NIOSH - National Institute for Occupational Safety and Health
PCI - Primary Containment Isolation
PCIOMR - Preconditioning Interim Operating Management Recommendations
RBCCW - Reactor Building Closed Cooling Water System
RBM - Rod Block Monitor
RCIC - Reactor Core Isolation Cooling System
RHRS - Residual Heat Removal System
RPS - Reactor Protection System
RWM - Rod Worth Minimizer
SBGTS - Standby Gas Treatment System
SBLC - Standby Liquid Control
SDC - Shutdown Cooling Mode of RHRS
SDV - Scram Discharge Volume
SRM - Source Range Monitor
TBCCW - Turbine Building Closed Cooling Water System
TIP - Traversing Incore Probe
TSC - Technical Support Center