

## NOTICE OF VIOLATION

Commonwealth Edison Company  
LaSalle County Station  
Units 1 and 2

Dockets No. 50-373, 50-374  
Licenses No. NPF-11, NPF-18

During an NRC inspection conducted on April 17 through May 24, 1996, three violations of NRC requirements were identified. In accordance with the "General Statement of Policy and Procedure for NRC Enforcement Actions," NUREG-1600, the violations are listed below:

1. Title 10 to the *Code of Federal Regulations*, Part 50, Appendix B, Criterion XVI, "Corrective Action," requires, in part, that measures be established to assure that conditions adverse to quality, such as failures, malfunctions, deficiencies, deviations, defective equipment, and non-conformances are promptly identified and corrected.

Contrary to the above,

- a. On April 6, 1996, a condition adverse to quality, specifically, a degraded support on the Reactor Core Isolation Cooling (RCIC) system, was identified by an operator and was not promptly corrected. The support was not evaluated by engineering until April 21 and not repaired until April 22, 1996.
- b. On December 26, 1990, a design deficiency was identified on the 2B Diesel Generator (DG), specifically, the motor on the motor-driven fuel pump was undersized, and a temporary alteration was made to the DG as a compensatory action. As of May 24, 1996, the design deficiency had not been corrected and the temporary alteration remained in place.

This is a Severity Level IV violation (Supplement I).

2. Title 10 to the *Code of Federal Regulations*, Part 50, Paragraph 50.59 requires, in part, that a licensee may make changes in the procedures as described in the safety analysis report providing a determination is made and documented, including the bases for the determination, that the change does not involve an unreviewed safety question.

Contrary to the above, the procedure for operation of the traversing incore probe (TIP) system was changed in 1991 in a manner different from described in the safety analysis report, in that the TIPs were allowed to remain in the drywell for up to 72 hours after operation in the reactor core, even though the safety analysis description stated that the penetration would be open an average of 15 hours per month. This change was authorized by a safety evaluation screening which did not recognize the existence of the 15 hour criterion and consequently did not include a documented basis for the determination that the change did not involve an unreviewed safety question.

This is a Severity Level IV violation (Supplement I).

3. 10 CFR 26.20 states written policies and procedures must address fitness for duty. Further, 10 CFR 26.20 requires that the license's fitness for duty policy address factors that could affect fitness for duty such as mental stress, fatigue, and illness.

The licensee's Fitness For Duty Policy states, in part, that it is the intent of the policy to provide reasonable assurance that nuclear plant personnel are not mentally or physically impaired in any way which adversely affects their ability to safely and competently perform their duties.

LaSalle Administrative Procedure 1100.25, "Testing for Cause," paragraph 8.a, requires any observed behavior of a contractor or vendor indicating degradation in performance, impairment, or change in behavior, be reported to the contractor's supervisor.

Contrary to the above, on April 27 and 28, 1996, two security officers observed behavior on the part of a third officer that indicated a degradation in performance, or a change in behavior, and failed to notify supervision of their observations in a timely manner. The observed behavior involved vandalism to company property (non-safety related) and verbal explanations for such actions.

This is a Severity IV violation (Supplement I).

Pursuant to the provisions of 10 CFR 2.201, Commonwealth Edison Company is hereby required to submit a written statement of explanation to the U.S. Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington, DC 20555 with a copy to the Regional Administrator, Region III, and a copy to the NRC Resident Inspector at the LaSalle facility, within 30 days of the date of the letter transmitting this Notice of Violation (Notice). This reply should be clearly marked as a "Reply to a Notice of Violation" and should include for each violation: (1) the reason for the violation, or, if contested, the basis for disputing the violation, (2) the corrective steps that have been taken and the results achieved, (3) the corrective steps that will be taken to avoid further violations, and (4) the date when full compliance will be achieved. Your response may reference or include previous docketed correspondence, if the correspondence adequately addresses the required response. If an adequate reply is not received within the time specified in this Notice, an order or a Demand for Information may be issued as to why the license should not be modified, suspended, or revoked, or why such other action as may be proper should not be taken. Where good cause is shown, consideration will be given to extending the response time.

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Because your response will be placed in the NRC Public Document Room (PDR), to the extent possible, it should not include any personal privacy, proprietary, or safeguards information so that it can be placed in the PDR without redaction. However, if you find it necessary to include such information, you should clearly indicate the specific information that you desire not to be placed in the PDR, and provide the legal basis to support your request for withholding the information from the public.

Dated at Lisle, Illinois  
this 19 day of August 1996