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September 13, 1996

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

ATTENTION: T. R. QUAY

SUBJECT: TABLE OF CONTENTS AND BINDER TABS FOR AP600 SHUTDOWN
ERG BACKGROUND DOCUMENTS

Reference 1: DCP/NRC0574, "AP600 Shutdown Lower Power Emergency Response
Guidelines and Background Documents," dated August 8, 1996.

Dear Mr. Quay:

Enclosed are binder tabs for sections of the background information for AP600 Shutdown Emergency Response Guidelines SDF-0.1, and SGD-1 through SDG-6 that were not transmitted with Reference 1. Those tabs should be inserted at the end of Background Book 2. The blue Shutdown Safety Status Trees tab should be followed by the white binder tabs separating SDF-0.1 and SDG-1 through SDG-6. Also enclosed is a revised Table of Contents, revised to reflect SDF-0.1 and SDG-1 through SDG-6.

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/nja
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EMERGENCY RESPONSE GUIDELINES TABLE OF CONTENTS

STATEMENT OF INTENT

OPTIMAL RECOVERY GUIDELINES

AE-0	Reactor Trip or Safety Injection
AES-0.1	Reactor Trip Response
AES-0.2	Natural Circulation Cooldown
AE-1	Loss of Reactor or Secondary Coolant
AES-1.1	Passive Safety Systems Termination
AES-1.2	Post LOCA Cooldown and Depressurization
AE-2	Faulted Steam Generator Isolation
AE-3	Steam Generator Tube Rupture
AECA-1.1	LOCA Outside Containment

STATUS TREES

AF-0.1	Subcriticality
AF-0.2	Core Cooling
AF-0.3	Heat Sink
AF-0.4	Integrity
AF-0.5	Containment
AF-0.6	Inventory

FUNCTION RESTORATION GUIDELINES

AFR-S.1	Response to Nuclear Power Generation/ATWS
AFR-S.2	Response to Loss of Core Shutdown
AFR-C.1	Response to Inadequate Core Cooling
AFR-C.2	Response to Degraded Core Cooling
AFR-C.3	Response to Saturated Core Cooling
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AFR-H.2	Response to Steam Generator Overpressure
AFR-H.3	Response to Steam Generator High Level
AFR-H.4	Response to Loss of Normal Steam Release Capabilities
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AFR-P.1	Response to Imminent Pressurized Thermal Shock Condition
AFR-P.2	Response to Anticipated Pressurized Thermal Shock Condition
AFR-Z.1	Response to High Containment Pressure
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AFR-Z.3	Response to High Containment Radiation Level
AFR-Z.4	Response to Low Containment Pressure
AFR-I.1	Response to High Pressurizer Level
AFR-I.2	Response to Low Pressurizer Level
AFR-I.3	Response to Voids in Reactor Vessel

SHUTDOWN SAFETY STATUS TREES

- SDF-0.1 Shutdown Safety Status Tree
- SDG-1 Response to Loss of RCS Inventory During Shutdown
- SDG-2 Response to Loss of RNS During Shutdown
- SDG-3 Response to High Containment Radiation During Shutdown
- SDG-4 Response to Increasing Nuclear Flux During Shutdown
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- SDG-6 Response to Unexpected RCS Temperature Changes During Shutdown

**EMERGENCY RESPONSE GUIDELINES
BACKGROUND INFORMATION BOOKS
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AES-0.1	Reactor Trip Response
AES-0.2	Natural Circulation Cooldown
AE-1	Loss of Reactor or Secondary Coolant
AES-1.1	Passive Safety Systems Termination
AES-1.2	Post Loss-of-Coolant Accident Cooldown and Depressurization
AE-2	Faulted Steam Generator Isolation
AE-3	Steam Generator Tube Rupture
AECA-1.1	Loss-of-Coolant Accident Outside Containment

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