

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Brunswick Steam Electric Plant Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 2 5	PAGE (3) 1 OF 0 1
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TITLE (4)
Automatic Isolations of the Unit 1 Reactor Core Isolation Cooling System

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0 3	0 9	8 5	8 5	0 1 4	0 0	0 4	0 3	8 5			0 5 0 0 0
DOCKET NUMBER(S) 0 5 0 0 0											

OPERATING MODE (9) 1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)									
POWER LEVEL (10) 0 1 6 1 0	<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 50.406(a)	<input checked="" type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)						
	<input type="checkbox"/> 20.406(a)(1)(i)	<input type="checkbox"/> 50.36(c)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(e)						
	<input type="checkbox"/> 20.406(a)(1)(ii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 365A)						
	<input type="checkbox"/> 20.406(a)(1)(iii)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)							
	<input type="checkbox"/> 20.406(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)							
	<input type="checkbox"/> 20.406(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)							

LICENSEE CONTACT FOR THIS LER (12)		TELEPHONE NUMBER	
NAME M. J. Pastva, Jr., Regulatory Technician		AREA CODE 9 1 1 9	NUMBER 4 1 5 7 1 - 1 2 1 3 1 1 5

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)											
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS		
X	B N	R T V	V 0 8 5	Yes							

SUPPLEMENTAL REPORT EXPECTED (14)			EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO					

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)

On 3-9-85, at 1615 and 1706, the Unit 1 Reactor Core Isolation Cooling (RCIC) System automatically isolated due to actuation of the RCIC equipment room ventilation differential high temperature instrument 1-E51-DTS-N601A. This shut the inboard steam isolation valve 1-E51-F007. A body-to-bonnet steam leak on RCIC System valve 1-E51-V20 caused localized high temperature conditions detected by N601A. Unit 1 was operating at 60 percent power. These events are revealed to the Unit 1 Control Operator through appropriate Control Room alarm annunciations.

The RCIC System was isolated and placed under clearance for repairs to 1-E51-V20. Appropriate repairs to V20, Velan Model No. W4-274B-2TS, were made and the RCIC System was returned to service on 3-13-85, at 1900. 1-E51-V20 is the instrument inboard root isolation valve to the high level transmitter of the RCIC System turbine steam supply inlet drain pot.

During the period the RCIC System was unavailable, the unit High Pressure Coolant Injection System was available to fulfill the RCIC System functions, if required.

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Carolina Power & Light Company

Brunswick Steam Electric Plant
P. O. Box 10429
Southport, NC 28461-0429
April 3, 1985

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SERIAL: BSEP/85-0571

NRC Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, DC 20555

BRUNSWICK STEAM ELECTRIC PLANT UNIT 1
DOCKET NO. 50-325
LICENSE NO. DPR-71
LICENSEE EVENT REPORT 1-85-014

Gentlemen:

In accordance with Title 10 to the Code of Federal Regulations, the enclosed Licensee Event Report is submitted. This report fulfills the requirement for a written report within thirty (30) days of a reportable occurrence and is in accordance with the format set forth in NUREG-1022, September 1983.

Very truly yours,

C. R. Dietz, General Manager
Brunswick Steam Electric Plant

MJP/smp/LETSMP

Enclosure

cc: Dr. J. N. Grace

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