

June 25, 1996

Mr. Charles D. Frizzle, President
Maine Yankee Atomic Power Company
329 Bath Road
Brunswick, ME 04011

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION--GENERIC LETTER 95-07, "PRESSURE
LOCKING AND THERMAL BINDING OF SAFETY-RELATED POWER-OPERATED GATE
VALVES," MAINE YANKEE (TAC NO. M93481)

Dear Mr. Frizzle:

On August 17, 1995, the NRC issued Generic Letter (GL) 95-07, "Pressure
Locking and Thermal Binding of Safety-Related Power-Operated Gate Valves."
This GL requests that licensees take actions to ensure that safety-related
power-operated gate valves that are susceptible to pressure locking or thermal
binding are capable of performing their safety function. The NRC staff has
reviewed and evaluated your response to GL 95-07 and additional information,
as discussed in the enclosure, is requested so that the staff may complete its
review. We request that you respond to this request within 30 days of receipt
of this letter.

Sincerely,



E. H. Trottier, Project Manager
Project Directorate I-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-309

Enclosure: Request for
Additional Information

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Maine Yankee Atomic Power Station

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REQUEST FOR ADDITIONAL INFORMATION

MAINE YANKEE RESPONSE TO GENERIC LETTER 95-07, "PRESSURE LOCKING AND THERMAL BINDING OF SAFETY-RELATED POWER-OPERATED GATE VALVES"

1. Attachment 1 of your submittal states that valves PR-M-16/17, Power Operated Relief Valve Block Valves, were modified with vent holes to relieve possible pressure locking. In Attachment 2, your submittal identifies valves PR-M-16/17 as solid wedge gate valves. Please clarify the actual design of these valves (flexible wedge or solid wedge gate).

In addition, your submittal states that in 1992, these valves were changed to "close on limit without wedging" to prevent thermal binding. Have you performed diagnostic testing that may support the assertion that the wedging thrust on these valves has been minimized? If so, please discuss this testing and provide a summary of the results for our review.

2. In Attachment 7, your submittal discusses planned and completed corrective actions taken in response to the pressure locking and thermal binding issue. The NRC staff believes you have been proactive in the resolution of this issue.

Through review of operational experience feedback, the staff is aware of instances in which other licensees have completed design or procedural modifications to preclude pressure locking or thermal binding, but these modifications may have had an adverse impact on plant safety due to incomplete or incorrect evaluation of their potential effects. Please describe evaluations and operator training that may have been conducted for each design or procedural modification to address potential pressure locking or thermal binding concerns.