NRC Form 366 (9-83)

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Limerick Generating Station - Unit 1	0 5 0 0 0 3 5 2	1 OF 0 2

TITLE (4)

Automatic Isolation of the Reactor Water Cleanup System

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
										0 5 0 0 0
1 2	1 2	8 4	8 4	- 0 3 4	- 0 0	0 1	1 1	8 5		0 5 0 0 0

OPERATING MODE (9)	POWER LEVEL (10)	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
4	0 0 0	20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
John C. Nagle, Engineer	AREA CODE 2 1 5 8 4 1 - 5 1 8 4

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
B	C E	T D S	R 2 7 8	Y					

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

Abstract: 84-034

On December 12, 1984, the reactor water cleanup (RWCU) system outboard isolation valve closed to the isolation position upon receiving a signal from the high differential temperature logic circuit of the reactor water cleanup portion of the Nuclear Steam Supply Shutoff System (NSSSS). The isolation signal was unexpectedly generated by setting the Temperature Differential Transmitter Switch TDTS-44-1N602S to the "READ" position. The isolation signal was cleared, and the reactor water cleanup system was returned to service.

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
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TEXT (If more space is required, use additional NRC Form 366A (17))

Description of the Event:

On December 12, 1984, at 5:20 a.m., prior to initial criticality, the reactor water cleanup system outboard isolation valve, HV-44-1F004 received a signal from the high differential temperature logic curcuit of the RWCU portion of NSSSS. HV-44-1F004 closed to the isolation position. After the high differential temperature isolation signal was cleared, the reactor water cleanup system was returned to normal operation at 5:57 a.m., the same morning.

Consequences of the Event:

The reactor water cleanup system isolated properly upon receiving the high differential temperature signal. There were no adverse consequences. In addition, reactor water chemistry, because of the short duration of the isolation, was not adversely affected.

Cause of the Event:

Setting the Temperature Differential Transmitter Switch TDTS-44-1N602S to the "READ" position caused a spurious isolation. A defect in the temperature differential transmitter switch is believed to be responsible for the spurious isolation signal.

Corrective Actions:

Information Tags have been placed on the inboard and outboard RWCU ambient temperature indicating panels in the Auxiliary Equipment Room. These tags state that "Before operating the differential temperature switches, notify the main control room that a RWCU isolation may result." A modification is being pursued to the "READ" circuit which will prevent inadvertent trips when using the "READ" switch on the temperature switch.

cc: Judge Helen F. Hoyt
Judge Jerry Harbour
Judge Richard F. Cole
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Judge Gary J. Edles
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January 11, 1985

Docket No. 50-352

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, DC 20555

SUBJECT: Licensee Event Report
Limerick Generating Station - Unit 1

This LER deals with the automatic isolation of the reactor water cleanup system. This event occurred prior to initial criticality.

Reference:	Docket No. 50-352
Report Number:	84-034
Revision Number:	00
Event Date:	December 12, 1984
Report Date:	January 11, 1985
Facility:	Limerick Generating Station P.O. Box A, Sanatoga, PA 19464

This LER is submitted pursuant to the requirements of 10CFR50.73 (a)(2)(iv).

Very truly yours,

W. T. Ullrich
Superintendent
Nuclear Generation Division

cc: Dr. Thomas E. Murley, Administrator, Region I, USNRC
J. T. Wiggins, Senior Site Inspector
See Service List

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