Mr. C. Lance Terry
Group Vice President, Nuclear
TU Electric
Energy Plaza
1601 Bryan Street, 12th Floor
Dallas, TX 75201-3411

June 18, 1996

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION - GENERIC LETTER 95-07, "PRESSURE LOCKING AND THERMAL BINDING OF SAFETY-RELATED POWER-OPERATED GATE VALVES," COMANCHE PEAK STEAM ELECTRIC STATION, UNITS 1 AND 2 (TAC NOS. M93449 AND M93450)

Dear Mr. Terry:

On August 17, 1995, the Nuclear Regulatory Commission (NRC) issued Generic Letter (GL) 95-07, "Pressure Locking and Thermal Binding of Safety-Related Power-Operated Gate Valves," to request that licensees take actions to ensure that safety-related power-operated gate valves that are susceptible to pressure locking or thermal binding are capable of performing their safety functions. The NRC staff is reviewing and evaluating your response to GL 95-07. Additional information, as discussed in the enclosure, is requested in order for the staff to complete its review. We request that you respond within 30 days.

Sincerely,
Original signed by
Phillip M. Ray, Acting Project Manager
Project Directorate IV-1
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-445 and 50-446

Enclosure: Request for Additional Information

cc w/encl: See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

June 18, 1996

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REQUEST FOR ADDITIONAL INFORMATION
COMANCHE PEAK STEAM ELECTRIC STATION, UNITS 1 AND 2, RESPONSE TO GENERIC LETTER 95-07, "PRESSURE LOCKING AND THERMAL BINDING OF SAFETY-RELATED POWER-OPERATED GATE VALVES"

1. Regarding valves 8000A/B, Pressurizer Power-Operated Relief Valves (PORV) Block Valves, the licensee's submittal states that the valves may be closed to isolate a leaking PORV, and that during a steam generator tube rupture event, reactor coolant system (RCS) pressure is less than the normal operating pressure of 2235 psig. Has the licensee evaluated this condition for potential depressurization-induced pressure locking? Please discuss this issue. It appears that the licensee has only considered the potential for thermally induced pressure locking.

2. Regarding valves 8716A/B, the licensee's submittal states that, since these valves are located downstream of the residual heat removal (RHR) heat exchanger, where the sump fluid has been cooled, and that since they are in a stagnant leg, not directly in contact with the fluid to the cold leg, then thermally induced pressure locking is not a concern. However, during the initial states of the recirculation phase of accident mitigation, water flowing in the RHR system may be significantly higher in temperature and may transfer heat to these valves. Please discuss this issue. Also, please provide any analysis completed, if applicable, for our review.

3. Through review of operational experience feedback, the staff is aware of instances where licensees have completed design or procedural modifications to preclude pressure locking or thermal binding which may have had an adverse impact on plant safety due to incomplete or incorrect evaluation of the potential effects of these modifications. Please describe evaluations and training for plant personnel that have been conducted for each design or procedural modification completed to address potential pressure locking or thermal binding concerns.