NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER)

APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

FACILITY NAME (1): Limerick Generating Station - Unit 1

DOCKET NUMBER (2): 0 5 0 0 0 3 5 2

PAGE (3): 1 OF 0 3

TITLE (4): Reactor Water Cleanup Isolation

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
1 2	1 1	8 4	8 4	- 0 3 2	- 0 0	0 1	1 0	8 5		0 5 0 0 0
										0 5 0 0 0

OPERATING MODE (9): 4

POWER LEVEL (10): 0 0 0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
20.405(a)(1)(i)	50.36(a)(1)	50.73(a)(2)(v)	73.71(c)
20.405(a)(1)(ii)	50.36(a)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME: John C. Nagle, Engineer/Supervisory

TELEPHONE NUMBER: AREA CODE 2 1 5 8 4 1 - 5 1 8 4

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)

		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

Abstract: 84-032

On December 11, 1984, the Reactor Water Cleanup (RWCU) inboard suction valve closed to the isolation position upon receiving a high ambient temperature trip signal. Cause of the event was personnel error. After the trip signal was cleared, the RWCU system was returned to service.

IE22
1/1

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Limerick Generating Station Unit 1	0 5 0 0 0 3 5 2	8 4	— 0 3 2	— 0 0	0 2 OF 0 3

TEXT (If more space is required, use additional NRC Form 366A's)

Description of the Event:

On December 11, 1984, at 3:40 p.m., prior to initial criticality, the Reactor Water Cleanup (RWCU) system inboard suction valve, HV-44-IF001, received an inadvertent high ambient temperature trip isolation signal. HV-44-IF001 isolated properly to the closed position. After the isolation signal was cleared, the RWCU system was returned to normal operation.

Consequences of the Event:

The portion of the RWCU system affected by the high ambient temperature signal isolated properly. There were no adverse consequences.

Cause of the Event:

At the time of the event, instrument and controls technicians were performing a surveillance test on a RWCU outboard logic high ambient temperature isolation switch. Since actuation of any one of these temperature switches causes the RWCU outboard suction valve, HV-44-IF004, to isolate closed, the surveillance test requires blocking HV-44-IF004 in the open position if RWCU is in service at the time of the test. HV-44-IF004 was blocked in the open position with its feed removed as required by the surveillance test.

Concurrent with the performance of the surveillance test referenced above, a second group of instrument and controls technicians was attempting to resolve a problem with another RWCU high ambient temperature isolation switch, TDTS-44-IN602N. This group believed that TDTS-44-IN602N was also in the outboard RWCU isolation logic, and the group verified that HV-44-IF004 was blocked open prior to start of work. However, TDTS-44-IN602N is actually in the inboard RWCU isolation logic. As a result, the inboard RWCU suction valve, HV-44-IF001, isolated to the closed position when TDTS-44-IN602N was actuated during checkout.

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Limerick Generating Station Unit 1	0 5 0 0 0 3 5 2	8 4	— 0 3 2	— 0 0	0 3 OF 0 3

TEXT (If more space is required, use additional NRC Form 366A's)

Corrective Actions:

The instrument and controls personnel involved with actuating the inboard logic isolation switch were reminded of the importance of reviewing the proper electrical schematics prior to starting work on safety-related systems.

PHILADELPHIA ELECTRIC COMPANY

2301 MARKET STREET

P.O. BOX 8699

PHILADELPHIA, PA. 19101

(215) 841-4000

January 10, 1985

Docket No. 50-352

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, DC 20555

SUBJECT: Licensee Event Report
Limerick Generating Station - Unit 1

This LER deals with the automatic isolation of the Reactor Water Cleanup system. This event occurred prior to initial criticality.

Reference:	Docket No. 352
Report Number:	84-032
Revision Number:	00
Event Date:	December 11, 1984
Report Date:	January 10, 1985
Facility:	Limerick Generating Station P.O. Box A, Sanatoga, PA 19464

This LER is submitted pursuant to the requirements of 10 CFR 50.73(a)(2)(iv).

Very truly yours,

W. T. Ullrich
Superintendent
Nuclear Generation Division

cc: Dr. Thomas E. Murley, Administrator, Region I, USNRC
J. T. Wiggins, Senior Site Inspector
See Service List

IE22
1/1

cc: Judge Helen F. Hoyt
Judge Jerry Harbour
Judge Richard F. Cole
Judge Christine N. Kohl
Judge Gary J. Edles
Judge Reginald L. Gotchy
Troy B. Conner, Jr., Esq.
Ann P. Hodgdon, Esq.
Mr. Frank R. Romano
Mr. Robert L. Anthony
Ms. Phyllis Zitzer
Charles W. Elliott, Esq.
Zori G. Ferkin, Esq.
Mr. Thomas Gerusky
Director, Penna. Emergency Management Agency
Angus Love, Esq.
David Wersan, Esq.
Robert J. Sugarman, Esq.
Martha W. Bush, Esq.
Spence W. Perry, Esq.
Jay M. Gutierrez, Esq.
Atomic Safety & Licensing Appeal Board
Atomic Safety & Licensing Board Panel
Docket & Service Section (3 Copies)
James Wiggins
Timothy R. S. Campbell